

LICENSEE EVENT REPORT

AO-S2-75-20

CONTROL BLOCK

[PLEASE PRINT ALL REQUIRED INFORMATION]

LICENSEE NAME 01 V A S P S 2										LICENSE NUMBER 00 - 000000 - 00										LICENSE TYPE 4 1 1 1 1					EVENT TYPE 0 1	
7	8	9	14	15	25	28	30	31	32	57	58	59	60	61	68	69	74	75	80							
CATEGORY 01 CONT			REPORT TYPE P O		REPORT SOURCE T L		DOCKET NUMBER 050 - 0281					EVENT DATE 1 2 1 9 7 5					REPORT DATE 0 1 0 2 7 6									
7	8	9	14	15	25	28	30	31	32	57	58	59	60	61	68	69	74	75	80							

EVENT DESCRIPTION

02 During normal operation of Unit No. 2 at 100 per cent power "C" steam generator pres-										7	8	9
03 sure indication-496-Channel IV failed high. This is a violation of section 3.7 of the										7	8	9
04 Technical Specifications. An immediate rampdown of 150 tWe per hour was initiated and										7	8	9
05 an instrument technician was dispatched to place the channel in the trip mode.										7	8	9
06 (AO-S2-75-20)										7	8	9

SYSTEM CODE 07 C C			CAUSE CODE C		COMPONENT CODE P I P E X X					PRIME COMPONENT SUPPLIER N		COMPONENT MANUFACTURER 2 9 9 9					VIOLATION Y	
7	8	9	10	11	12	17	43	44	47	48								

CAUSE DESCRIPTION

08 Examination of the failed transmitter revealed the sensing line to be frozen. This										7	8	9
09 condition was caused by abnormally cold outside air being drawn through a set of in-										7	8	9
10 take louvers (located adjacent to the sensing line in the safeguards building (con't))										7	8	9

FACILITY STATUS 11 L			% POWER 1 0 0			OTHER STATUS N/A			METHOD OF DISCOVERY A		DISCOVERY DESCRIPTION N/A				
7	8	9	10	12	13	44	45	46	80						

FIRM OF ACTIVITY RELEASED 12 Z			CONTENT OF RELEASE Z			AMOUNT OF ACTIVITY N/A			LOCATION OF RELEASE N/A				
7	8	9	10	11	44	45	80						

PERSONNEL EXPOSURES

NUMBER 13 0 0 0			TYPE Z		DESCRIPTION N/A				
7	8	9	11	12	13	80			

PERSONNEL INJURIES

NUMBER 14 0 0 0			DESCRIPTION N/A				
7	8	9	11	12	80		

OFFSITE CONSEQUENCES

15 N/A										7	8	9
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LOSS OR DAMAGE TO FACILITY

TYPE 16 Z			DESCRIPTION N/A						
7	8	9	10	80					

PUBLICITY

17 N/A										7	8	9
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8403090335 760102
 PDR ADOCK 05000281
 S PDR

ADDITIONAL FACTORS

18 The probability of a similar failure of another transmitter has been eliminated by										7	8	9
19 sealing the intake louvers and will be further removed by the installation (con't)										7	8	9

NAME: T. L. Baucom

PHONE: (804) 357-3184

CAUSE DESCRIPTION (con't)

wall) and over the sensing line. The louvers were closed and sealed allowing the line to thaw and the transmitter to be returned to normal service.

ADDITIONAL FACTORS (con't)

of heat tracing on the sensing lines which are located adjacent to the ventilation louvers.

The ability of the safety injection system to perform its function was in no way impaired by this failure. Although the transmitter failed in the nonconservative direction (drifted high) it was only in this condition for a brief period of time before it was placed in the trip mode. In addition the other two remaining channels were in proper working order and could have provided a safety injection signal, if required, since only two out of three channels are required to initiate a safety injection.

The health and safety of the general public was in no way endangered since the safety injection system was able to perform its intended function at all times during this occurrence.

This is a similar occurrence to that noted on Unit No. 1 on December 19, 1975.

VIRGINIA ELECTRIC AND POWER COMPANY
RICHMOND, VIRGINIA 23261

January 15, 1976



Mr. Norman C. Moseley, Director
Office of Inspection and Enforcement
United States Nuclear Regulatory Commission
Region II - Suite 818
230 Peachtree Street, Northwest
Atlanta, Georgia 30303

Docket No. 50-281
License No. DPR-37

Dear Mr. Moseley:

IE FILE COPY

Pursuant to Surry Power Station Technical Specification 6.6.B.1,
the Virginia Electric and Power Company hereby submits forty (40) copies
of Abnormal Occurrence Report No. AO-S2-75-20.

The substance of this report has been reviewed by the Station Nuclear
Safety and Operating Committee and will be placed on the agenda for the
next meeting of the System Nuclear Safety and Operating Committee.

Very truly yours,

C. M. Stallings

C. M. Stallings
Vice President-Power Supply
and Production Operations

Enclosures
40 copies of AO-S2-75-20

cc: Mr. Robert W. Reid

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