

LICENSEE EVENT REPORT

AO-SI-73-17

CONTROL BLOCK

(PLEASE PRINT ALL REQUIRED DATA)

7	8	9	0	1	2	3	4	5	6	7	8	9	0	1	2	3	4	5	6	7	8	9	0																																				
V A S P S I												0 0 - 0 0 0 0 0 - 0 0												4 1 1 1 0																																			
CONT P O												T L												0 5 0 - 0 2 8 0												1 2 3 0 7 5												0 1 0 9 7 6											

EVENT DESCRIPTION

07 While operating at 100 per cent power a component cooling water/service water heat exchanger was opened for cleaning. Two tubes that had been previously plugged were noted leaking indicating that an inadvertent release of low level radioactive water to the James River had occurred. The leak rate was measured to be .008 gal per minute. The heat exchanger was isolated, preventing further release. This didn't

7	8	9	0	1	2	3	4	5	6	7	8	9	0	1	2	3	4	5	6	7	8	9	0
SYSTEM CODE				CAUSE CODE				COMPONENT CODE				PRIME COMPONENT SUPPLIER				COMPONENT MANUFACTURER				VIOLATION			
W B				F				H T E X C H				A				Y O 3 0				Y			

CAUSE DESCRIPTION

08 A Yuba Heat Transfer Division 51-449 heat exchanger had 36 tubes plugged as the result of eddy current tests. Two plugs worked partly out during operation and caused leakage from component cooling water to service water. The plugs were (don't)

7	8	9	0	1	2	3	4	5	6	7	8	9	0	1	2	3	4	5	6	7	8	9	0
FACILITY STATUS				% POWER				OTHER STATUS				METHOD OF DISCOVERY				DISCOVERY DESCRIPTION							
E				1 0 0				N/A				B				Routine cleaning							
FORM OF ACTIVITY RELEASED				CONTENT OF RELEASE				AMOUNT OF ACTIVITY				LOCATION OF RELEASE											
M				M				Unknown				Turbine building, James River											

PERSONNEL EXPOSURES

7	8	9	0	1	2	3	4	5	6	7	8	9	0
NUMBER				TYPE				DESCRIPTION					
0 0 0				Z				N/A					

PERSONNEL INJURIES

7	8	9	0	1	2	3	4	5	6	7	8	9	0
NUMBER				DESCRIPTION									
0 0 0				N/A									

OFFSITE CONSEQUENCES

15 N/A

LOSS OF DAMAGE TO FACILITY

7	8	9	0	1	2	3	4	5	6	7	8	9	0
TYPE				DESCRIPTION									
Z				N/A									

PUBLICITY

17 N/A

ADDITIONAL FACTORS

7	8	9	0	1	2	3	4	5	6	7	8	9	0

8403090320 760109
PDR ADDOCK 05000280
S PDR

7	8	9	0	1	2	3	4	5	6	7	8	9	0

NAME: E. M. Sweeney, Jr.

PHONE (801) 357-1111

EVENT DESCRIPTION (con't)

is contrary to Technical Specification 1.0.1.3.

CAUSE DESCRIPTION (con't)

replaced. Two other similar heat exchangers have experienced tube leakage (AO-SI-74-13, AO-SI-74-16). The three remaining heat exchangers were opened and inspected for tube leaks and none were found.

A design change is presently in progress to improve the radiation monitor system for service water leaving the component cooling water heat exchangers. Until it is completed a daily sample will be taken of service water for activity.

Based on the measured release rate the prorated average for all radionuclides released was 1.66×10^{-4} per cent MPC. The heat exchanger was last inspected on September 23, 1975. The estimate for total curies released is based on a leak rate of .008 gal per minute for the 98 days since the last inspection. This value is .018 curies. Because of these very low values for per cent MPC and total curies the health and safety of the general public were not affected by this occurrence.

VIRGINIA ELECTRIC AND POWER COMPANY
RICHMOND, VIRGINIA 23261

January 13, 1976



Mr. Norman C. Moseley, Director
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Region II - Suite 818
230 Peachtree Street, Northwest
Atlanta, Georgia 30303

Serial No. 354
PO&M/ALH:clw

Docket No. 50-280
License No. DPR-32

Dear Mr. Moseley:

Pursuant to Surry Power Station Technical Specification 6.6.B.1, the Virginia Electric and Power Company hereby submits forty (40) copies of Abnormal Occurrence Report No. AO-S1-75-30.

The substance of this report has been reviewed by the Station Nuclear Safety and Operating Committee and will be placed on the agenda for the next meeting of the System Nuclear Safety and Operating Committee.

Very truly yours,

C. M. Stallings

C. M. Stallings
Vice President-Power Supply
and Production Operations

Enclosure

40 copies of AO-S1-75-30

cc: Mr. Robert W. Reid, Chief
Operating Reactors Branch 4

Mr. Bryce P. Schofield, Director
Bureau of Industrial Hygiene
Commonwealth of Virginia