

LICENSEE EVENT REPORT

USRE-S2-76-12

CONTROL BLOCK

(PLEASE PRINT ALL REQUIRED INFORMATION)

LICENSEE NAME										LICENSE NUMBER										LICENSE TYPE										EVENT TYPE																													
[01] V A S P S 2										[0] 0 - [0] 0 0 0 0 0 - [0] 0										[4] 1 1 1 0										[0] 3																													
7 8 9 14 15 25 26 30 31 32																																																											
CATEGORY										REPORT TYPE										REPORT SOURCE										DOCKET NUMBER										EVENT DATE										REPORT DATE									
[01] CON'T [P] 0										[L] [L]										[0] 5 0 - [0] 2 8 1										[0] 8 1 3 7 6										[0] 9 0 3 7 6																			
7 8 57 58 59 60 61 68 69 74 75 80																																																											

EVENT DESCRIPTION

[02] During normal operation a routine sample of "C" safety injection accumulator indicated																																																																															
[03] 1893 ppm boron (Time: 1600). This is contrary to the Technical Specification 3.3.A.2																																																																															
[04] limit of 1950 ppm boron. Accumulator "C" was declared inoperable per Technical Speci-																																																																															
[05] fication 3.3.B and isolated. After recirculation with the RWST, the boron concentra-																																																																															
[06] tion increased to 2116 ppm (Time: 1845). A power rampdown was not necessary (con't)																																																																															
80																																																																															

SYSTEM CODE										CAUSE CODE										COMPONENT CODE										PRIME COMPONENT SUPPLIER										COMPONENT MANUFACTURER										VIOLATION									
[07] P C										[E]										[V] A L V E X										[A]										[0] 0 2 0										[Y]									
7 8 9 10 11 12 17 43 44 47 48																																																											

CAUSE DESCRIPTION

[08] Leaking of check valves (2-SI-147) and (2-SI-145) caused dilution of the accumulator's																																																																															
[09] boron concentration from the "C" primary loop. Surveillance of the accumulator's																																																																															
[10] boron concentration has been increased to weekly and the accumulator is re- (con't)																																																																															
80																																																																															

FACILITY STATUS										% POWER										OTHER STATUS										METHOD OF DISCOVERY										DISCOVERY DESCRIPTION									
[11] E										[1] 0 0										N/A										[B]										N/A									
7 8 9 10 12 13 44 45 46 80																																																	
FORM OF ACTIVITY RELEASED										CONTENT OF RELEASE										AMOUNT OF ACTIVITY										LOCATION OF RELEASE																			
[12] Z										[Z]										N/A										N/A																			
7 8 9 10 11 44 45 80																																																	

PERSONNEL EXPOSURES

NUMBER										TYPE										DESCRIPTION									
[13] 0 0 0										[Z]										N/A									
7 8 9 11 12 13 80																													

PERSONNEL INJURIES

NUMBER										DESCRIPTION									
[14] 0 0 0										N/A									
7 8 9 11 12 80																			

OFFSITE CONSEQUENCES

[15] N/A																																																																															
80																																																																															

LOSS OR DAMAGE TO FACILITY

TYPE										DESCRIPTION									
[16] Z										N/A									
7 8 9 10 80																			

PUBLICITY

[17] N/A										8403090156 760903 PDR ADOCK 05000281 S PDR									
7 8 9 80																			

ADDITIONAL FACTORS

[18] A similar reportable event was experienced with the "B" safety injection accumulator																																																																															
80																																																																															
[19] of Unit No. 1 (USRE-S1-76-08) (July 21, 1976), and analogous correction actions (con't)																																																																															
80																																																																															

NAME: Tyndall L. Baucom

PHONE: (804) 357-3184

EVENT DESCRIPTION (con't)

because the accumulator was isolated for a period less than four hours, as permitted by Technical Specification 3.3.B. This event is reportable per Technical Specification 6.6.2.b(1). (USRE-S2-76-12)

CAUSE DESCRIPTION (con't)

circulated with the RWST when necessary.

During the upcoming refueling the check valves will be inspected and repaired as necessary.

The total effect of this boric acid dilution would not have influenced shutdown capability at this point in core life. Any accident analyzed in the safety analysis would not have been affected by the small boron dilution.

ADDITIONAL FACTORS (con't)

were recommended. Since the safety injection system maintained its capabilities, the health and safety of the general public are not affected.

VIRGINIA ELECTRIC AND POWER COMPANY
RICHMOND, VIRGINIA 23261

September 10, 1976

Mr. Norman C. Moseley, Director
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Region II - Suite 818
230 Peachtree Street, Northwest
Atlanta, Georgia 30303

Serial No. 226
PO&M/ALH:clw

Docket No. 50-281
License No. DPR-37

Dear Mr. Moseley:

Pursuant to Surry Power Station Technical Specification 6.6.2, the Virginia Electric and Power Company hereby submits a copy of Reportable Occurrence No. USRE-S2-76-12.

The substance of this report has been reviewed by the Station Nuclear Safety and Operating Committee and will be placed on the agenda for the next meeting of the System Nuclear Safety and Operating Committee.

Very truly yours,

C. M. Stallings

C. M. Stallings
Vice President-Power Supply
and Production Operations

Enclosure

cc: Mr. Robert W. Reid, Chief
Operating Reactors Branch 4
(40 copies USRE-S2-76-12)