

USRE-S2-76-15

(PLEASE PRINT ALL REQUIRED INFORMATION)

EVENT	DESCRIPTION
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SYSTEM CODE		CAUSE CODE	COMPONENT CODE						COMPONENT SUPPLIER	COMPONENT MANUFACTURER				VIOLATION
S	H	E	P	I	P	E	X	X	A	Z	9	9	9	N
9	10	11	12					17	43	44			47	48

08	The apparent cause of failure was chloride stress corrosion in Type 304 stainless	80
09	steel piping. Further investigation is being performed on Unit No. 1 and Unit No. 2,	80
10	since both units utilize a similar arrangement for the recirculation spray system (cont)	80

11	FACILITY STATUS			% POWER			OTHER STATUS			METHOD OF DISCOVERY			DISCOVERY DESCRIPTION																
7 8	G			0 0 0			N/A			C			N/A																
	9			10 12			13			44			45 46 80																
12	FORM OF ACTIVITY RELEASED			CONTENT OF RELEASE			AMOUNT OF ACTIVITY			LOCATION OF RELEASE																			
7 8	Z			Z			N/A			N/A																			
	9			10 11			44			45 80																			

NUMBER				TYPE	DESCRIPTION
1	3	0	0	0	Z N/A

NUMBER				DESCRIPTION	
1	4	0	0	0	N/A

15	N/A
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TYPE			DESCRIPTION
16	Z		N/A

17	N/A	8403080444 781117	
7	8	PDR ADOCK 05000281	
	9	S PDR	RO

18 The health and safety of the general public were not affected.

19 89 80

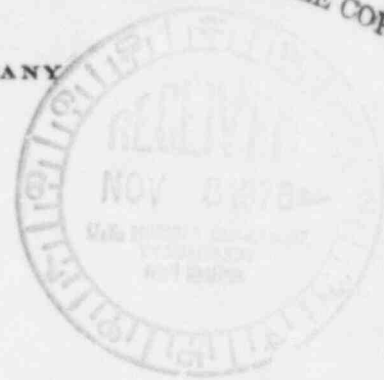
PHONE: (804) 357-3184

CAUSE DESCRIPTION (con't)

pipng. The extent of affected piping, the degree of damage, and the necessary repairs have not been completely evaluated at this time, and will be the subject of a future update report. Repairs as necessary will be accomplished prior to placing the units back into service.

VIRGINIA ELECTRIC AND POWER COMPANY  
RICHMOND, VIRGINIA 23261

November 5, 1976



Mr. Norman C. Moseley, Director  
Office of Inspection and Enforcement  
U.S. Nuclear Regulatory Commission  
Region II - Suite 818  
230 Peachtree Street, Northwest  
Atlanta, Georgia 30303

Serial No. 308  
PO&M/ALH:clw

Docket No. 50-281  
License No. DPR-37

Dear Mr. Moseley:

Pursuant to Surry Power Station Technical Specification 6.6.B.2, the Virginia Electric and Power Company hereby submits a copy of Licensee Event Report USRE-S2-76-15.

The substance of this report has been reviewed by the Station Nuclear Safety and Operating Committee and will be placed on the agenda for the next meeting of the System Nuclear Safety and Operating Committee.

Very truly yours,

*C. M. Stallings*

C. M. Stallings  
Vice President-Power Supply  
and Production Operations

Enclosure

cc: Mr. Robert W. Reid, Chief  
Operating Reactors Branch 4  
(40 copies USRE-S2-76-15)

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