

# LICENSEE EVENT REPORT

USRE-S2-76-16

CONTROL BLOCK

(PLEASE PRINT ALL REQUIRED INFORMATION)

LICENSEE NAME 01 V A S P S 2														LICENSE NUMBER 00-000000-00														LICENSE TYPE 41110						EVENT TYPE 03													
CATEGORY 01 CONT M T														REPORT TYPE L				REPORT SOURCE L				DOCKET NUMBER 050-0281														EVENT DATE 123076						REPORT DATE 011477					

## EVENT DESCRIPTION

02 As a result of a routine chemical analysis during steady operation, it was found that																																																																															
03 the Boron Injection Tank (BIT) boron concentration was 11.4% by weight, which is below																																																																															
04 the 11.5% low limit set forth in Technical Specification 3.3.A.3. An immediate																																																																															
05 reactor ramp down was begun until the concentration was again within the specifications																																																																															
06 At the time of the dilution, the Boron Injection Tank (BIT) was on recirc with "C"																																																																															

SYSTEM CODE 07 P C				CAUSE CODE E				COMPONENT CODE V A L V E X										PRIME COMPONENT SUPPLIER A				COMPONENT MANUFACTURER D O 2 0										VIOLATION Y			
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## CAUSE DESCRIPTION

08 The dilution of the BIT was most probably caused by leakage through one or both of																																																																															
09 the BIT inlet valves (MOV-2867A and B). These valves are normally closed and open																																																																															
10 automatically on a safety injection signal. After the required concentration was																																																																															

FACILITY STATUS 11 E				% POWER 097				OTHER STATUS NA				METHOD OF DISCOVERY B				DISCOVERY DESCRIPTION NA			
FORM OF ACTIVITY RELEASED 12 Z				CONTENT OF RELEASE Z				AMOUNT OF ACTIVITY NA				LOCATION OF RELEASE NA							

## PERSONNEL EXPOSURES

NUMBER 13 000				TYPE Z				DESCRIPTION NA			
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## PERSONNEL INJURIES

NUMBER 14 000				DESCRIPTION NA			
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## OFFSITE CONSEQUENCES

15 NA																																																																															
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## LOSS OR DAMAGE TO FACILITY

TYPE 16 Z				DESCRIPTION NA			
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## PUBLICITY

17 NA				8403080435 770114 PDR ADOCK 05000281 S PDR			
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## ADDITIONAL FACTORS

18 Because the capabilities of the Safety Injection System were not impaired by this																																																																															
19 minimal dilution of the BIT, the health and safety of the general public were not																																																																															

NAME: T. L. Baucom

PHONE: (804) 357-3184

EVENT DESCRIPTION (CONTINUED)

Boric Acid Storage Tank (BAST). Boric acid was then batched to "C" BAST to bring up the boron concentration to 11.9% in the BIT. In addition, the surveillance frequency of the BIT was increased until the source of the dilution was terminated. This event is reportable per Technical Specification 6.6.2.b(2) (USRE-S2-76-16).

CAUSE DESCRIPTION (CONTINUED)

obtained in the BIT, the MOV's were closed by means of both manual and electrical forces. A 1/4 inch stem movement was observed. Since this closure there has been no apparent dilution of the BIT. To provide long term corrective action, the valves' stroke will be checked during the next cold shutdown to insure positive shutoff.

ADDITIONAL FACTORS (CONTINUED)

affected.

VIRGINIA ELECTRIC AND POWER COMPANY  
RICHMOND, VIRGINIA 23261

1E FILE COPY

January 19, 1977



Mr. Norman C. Moseley, Director  
Office of Inspection and Enforcement  
U. S. Nuclear Regulatory Commission  
Region II - Suite 818  
230 Peachtree Street, Northwest  
Atlanta, Georgia 30303

Serial No. 003  
PO&M/ALH:dgt

Docket No. 50-281  
License No. DPR-37

Dear Mr. Moseley:

Pursuant to Surry Power Station Technical Specification 6.6.2, the Virginia Electric and Power Company hereby submits a copy of Reportable Occurrence No. USRE-S2-76-16.

The substance of this report has been reviewed by the Station Nuclear Safety and Operating Committee and will be placed on the agenda for the next meeting of the System Nuclear Safety and Operating Committee.

Very truly yours,

*C. M. Stallings*  
C. M. Stallings  
Vice President-Power Supply  
and Production Operations

Enclosure

cc: Mr. Robert W. Reid, Chief  
Operating Reactors Branch 4  
(40 copies USRE-S2-76-16)