

LICENSEE EVENT REPORT

CONTROL BLOCK:

[PLEASE PRINT ALL REQUIRED INFORMATION]

LICENSEE NAME		LICENSE NUMBER										LICENSE TYPE					EVENT TYPE						
01	V	A	S	P	S	2	0	0	-	0	0	0	0	-	0	0	4	1	1	1	0	0	1
7	8	9	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32		

CATEGORY		REPORT TYPE		REPORT SOURCE		DOCKET NUMBER										EVENT DATE					REPORT DATE				
01	CONT	P	O	T	L	0	5	0	-	0	2	8	1	0	5	1	1	7	6	0	5	2	0	7	6
7	8	57	58	59	60	61	62	63	64	65	66	67	68	69	70	71	72	73	74	75	76	77	78	79	80

EVENT DESCRIPTION

02	While removing a "pie" shaped section of the 7th tube support plate in the "2A" steam																								80
03	generator during the current refueling outage, small cracks and radial growth were ob-																								80
04	served on the periphery of the subject plate. Further investigation is in progress to																								80
05	insure that the structural integrity of the support plate is still satisfactory. This																								80
06	event is reportable per Technical Specification 6.6.2.a(9). (AO-S2-76-02)																								80

SYSTEM CODE		CAUSE CODE		COMPONENT CODE								PRIME COMPONENT SUPPLIER		COMPONENT MANUFACTURER				VIOLATION	
07	C	C	E	Z	Z	Z	Z	Z	Z	N	W	1	2	0	N				
7	8	9	10	11	12	13	14	15	16	17	43	44	45	46	47	48			

CAUSE DESCRIPTION

08	The tube denting and apparent support plate distortion are believed to be related to																								80
09	the change from phosphate to AVT chemistry. Investigative actions include: a finite																								80
10	element stress analysis of the tube support plate including cracks, analysis (Cont'd).																								80

FACILITY STATUS		% POWER			OTHER STATUS					METHOD OF DISCOVERY		DISCOVERY DESCRIPTION									
11	H	0	0	0	N/A	C	N/A														
7	8	9	10	11	12	13	44	45	46	80											

FORM OF ACTIVITY RELEASED		CONTENT OF RELEASE		AMOUNT OF ACTIVITY										LOCATION OF RELEASE									
12	Z	Z	N/A																				
7	8	9	10	11	80																		

PERSONNEL EXPOSURES

NUMBER			TYPE		DESCRIPTION										
13	0	0	0	Z	N/A										
7	8	9	11	12	13	80									

PERSONNEL INJURIES

NUMBER			DESCRIPTION											
14	0	0	0	N/A										
7	8	9	11	12	80									

OFFSITE CONSEQUENCES

15	N/A																								80
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LOSS OR DAMAGE TO FACILITY

TYPE		DESCRIPTION																						
16	Z	N/A																						
7	8	9	80																					

PUBLICITY

17	N/A																								80
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ADDITIONAL FACTORS

18																									80
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8403070416 760520
PDR ADOCK 05000281
S PDR

19																									80
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NAME: E. M. Sweeney, Jr.

PHONE: (804) 357-3184

CAUSE DESCRIPTION (Continued)

of a postulated steam break accident with the potential of loose metal segments in the generator, and analysis of ~~post~~-LOCA and post-steam break tube integrity under blowdown loads with cracked tube support plates.

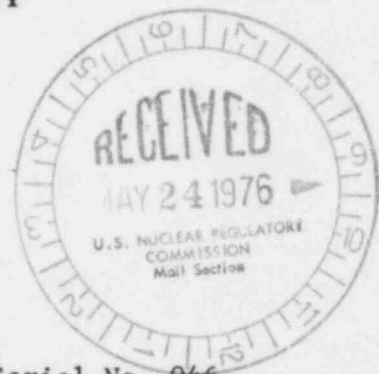
The configuration of the tube support plate/tube bundle and the corrosion mechanism leading to denting result in the development of compressive forces on the tubes and the plate itself. These forces tend to lock the tubes and plate firmly together even in the presence of cracking. These compressive forces have been observed during previous tube pulling and support plate segment removal operations when very large forces were required to remove the specimens.

Similar conditions were observed in tube support plates of other Westinghouse PWR's. Therefore this problem is considered generic in nature. The NRC Division of Reactor Licensing will be kept informed on this matter until the issue is closed.

Since the integrity of the steam generator tubes and tube support plate remain intact, the health and safety of the general public are not affected.

VIRGINIA ELECTRIC AND POWER COMPANY
RICHMOND, VIRGINIA 23261

May 21, 1976



Mr. Norman C. Moseley, Director
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Region II - Suite 818
230 Peachtree Street, Northwest
Atlanta, Georgia 30303

Serial No. 046
PO&M/ALH:jlf

Docket No. 50-281
License No. DPR-37

Dear Mr. Moseley:

Pursuant to Surry Power Station Technical Specification 6.6.2, the Virginia Electric and Power Company hereby submits a copy of Reportable Occurrence No. AO-S2-76-02.

The substance of this report has been reviewed by the Station Nuclear Safety and Operating Committee and will be placed on the agenda for the next meeting of the System Nuclear Safety and Operating Committee.

Very truly yours,

C. M. Stallings

C. M. Stallings
Vice President-Power Supply
and Production Operations

Enclosure

cc: Mr. Robert W. Reid, Chief (40)
Operating Reactors Branch 4

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