

CONTROL BLOCK:

2

6

LICENSE NUMBER

LICENSE
TYPE

EVENT
TYPE

NAME				LICENSE NUMBER								TYPE				TYPE		
01	C	O	F	S	V	I	00	-	00	00	00	-	00	41	1	20	01	
7	8	9				14	15						25	26		30	31	32

01 CONT		CATEGORY P0	REPORT TYPE T	REPORT SOURCE L	DOCKET NUMBER 050-0267				EVENT DATE 111475				REPORT DATE				
7	8	57	58	59	60	61			68	69			74	75			80

02 While shutdown with both circulators in one loop self-turbinizing,
03 one circulator tripped due to "loss of bearing water". The
04 other circulator remained operable. No reason for the trip
05 could be found.
06 (A/O 75/25)

SYSTEM CODE		CAUSE CODE	COMPONENT CODE					PRIME COMPONENT SUPPLIER	COMPONENT MANUFACTURER			VIOLATION		
07	CB	F	B	R	G	W	T	R	N	G	3	0	5	Y
7	8	9	10						43	44				48

09 The cause of the trip was a perturbation in the loop
09 two bearing water system. Perturbation cause is
10 unknown.

FACILITY STATUS		% POWER		OTHER STATUS		METHOD OF DISCOVERY		DISCOVERY DESCRIPTION	
11	G	000		N/A		d		NRC as reportable	
7	8	9	10	11	12	13	44	45	46
FORM OF ACTIVITY RELEASED		CONTENT OF RELEASE		AMOUNT OF ACTIVITY				LOCATION OF RELEASE	
12	Z	Z		N/A			N/A		
7	8	9	10	11	12	13	44	45	46

NUMBER				TYPE	DESCRIPTION
13	000	Z	N/A		

NUMBER				DESCRIPTION	
1	4	0	0	0	N/A

15 | N/A

TYPE			DESCRIPTION
16	<input checked="" type="checkbox"/>		N/A

12 N/A

1a N/A S PDR

19 | 7 8 9 | *all : 11 : 6 d' 1* | 90

NAME: _____

PHONE: 571-7511 Ext. 456

San Francisco Operations Office
ERDA
1333 Broadway
Oakland, California 94612

Attn: Manager - - - - - 2*
California Patent Group - - - - - 1*

Division of Reactor Research
and Development
ERDA

Washington, D.C. 20545

Attn: Director - - - - - 1*
Asst. Dir. for Gas Cooled Reactor Projects - - - - - 2*
Asst. Dir. for Engrg. and Technology - - - - - 1*
Asst. Dir. for Reactor Safety - - - - - 1*

ERDA-SCRPO-SD - - - - - 1*
P. O. Box 81325
San Diego, California 92138

Project Engineer - - - - - 1*
ERDA - SCRPO
P. O. Box 1446
Canoga Park, California 91304

Address +++ Director - - - - - 1
Letter ERDA - SCRPO
To P. O. Box 1446
Canoga Park, California 91304

Technical Information Center - - - - - 3*
ERDA
P. O. Box 62
Oak Ridge, Tennessee 37830

Address +++ Mr. E. Morris Howard, Director - - - - - 1 (copy of Licensee Event
P Letter Region IV, Report)
To Office of Inspection & Enforcement
Nuclear Regulatory Commission
611 Ryan Plaza Drive
Suite 1000
Arlington, Texas 76012

Director of Nuclear Reactor Regulation - - - - - 1 (copy of Howard's letter and
Attn: Mr. Roger S. Boyd, Acting Director maillogram)
Division of Reactor Licensing
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

Address +++ Office of Information & Program Control - - - - - 1 (Original of LER)
P Letter U. S. Nuclear Regulatory Commission
To Washington, D.C. 20555

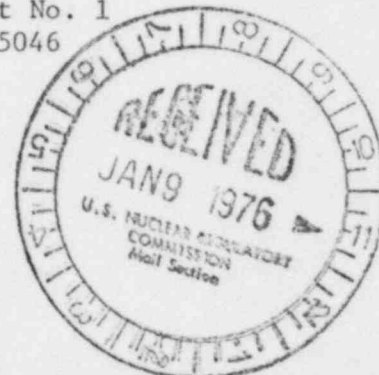
Address +++ John M. Waage, Project Manager - - - - - 10*
FFLG General Atomic Company
Letter P. O. Box 81608
To San Diego, California 92138

Secretary, NFSC, Belleview - - - - - 1
Vice President, Engineering & Planning, 39th Avenue Engineering - - - - 1
Manager of Production (Chairman, NFSC), Belleview - - - - - 1
Director, Quality Assurance - - - - - 1
Technical Services Supervisor - - - - - 1
Superintendent, Nuclear Production - - - - - 1
Superintendent, Operations - - - - - 1
Training Coordinator - - - - - 1
PORC Committee - - - - - 1
Technical Services Files - - - - - 1
Training Review - - - - - 6



Public Service Company of Colorado
P. O. Box 361, Platteville, Colorado 80651

December 31, 1975
Fort St. Vrain
Unit No. 1
P-75046



Mr. E. Morris Howard, Director
Nuclear Regulatory Commission
Region IV
Office of Inspection and Enforcement
Suite 1000
Arlington, Texas 76012

Ref: Facility Operating License
No. DPR-34

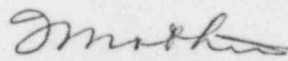
Docket No. 50-267

Dear Mr. Howard:

Enclosed please find a copy of Abnormal Occurrence Report No. 50-267/75/25, Final, submitted per the requirements of the Technical Specifications.

Also, please find enclosed one copy of the Licensee Event Report for Abnormal Occurrence Report No. 50-267/75/25.

Very truly yours,


for Frederic E. Swart
Superintendent, Nuclear Production
Fort St. Vrain Nuclear
Generating Station

FES/alk

cc: Mr. Roger S. Boyd

REPORT DATE: December 31, 1975

ABNORMAL OCCURRENCE 75/25

OCCURRENCE DATE: December 22, 1975

Page 1 of 3

FORT ST. VRAIN NUCLEAR GENERATING STATION
PUBLIC SERVICE COMPANY OF COLORADO
P. O. BOX 361
PLATTEVILLE, COLORADO 80651

REPORT NO. 50-267/75/25

Final

IDENTIFICATION OF
OCCURRENCE:

Helium circulator C-2103 tripped. This is identified as an abnormal occurrence per Section 2.1, paragraph (F), of the Fort St. Vrain Technical Specifications.

This problem occurred November 14, 1975, and in discussion with the Nuclear Regulatory Commission on December 22, 1975, it was agreed to report this incident as an abnormal occurrence.

CONDITIONS PRIOR
TO OCCURRENCE:

<u>Steady State Power</u>	<u>Routine Shutdown</u>
<u>Hot Shutdown</u>	<u>Routine Load Change</u>
<u>X Cold Shutdown</u>	<u>Other (specify)</u>
<u>Refueling Shutdown</u>	<u></u>
<u>Routine Startup</u>	<u></u>

The major plant parameters at the time of the event were as follows:

Power	RTR	<u>0</u>	MWth
	ELECT	<u>0</u>	MWe
Secondary Coolant	Pressure	<u>1,333</u>	psig
	Temperature	<u>214</u>	°F
	Flow	<u>49,000</u>	#/hr.
Primary Coolant	Pressure	<u>333</u>	psig
	Temperature	<u>214</u>	°F Core Inlet
		<u>207</u>	°F Core Outlet
	Flow	<u>N/A*</u>	#/hr.

*Loop 2 circulators, C-2103 and C-2104, self-turbining.

DESCRIPTION OF
OCCURRENCE:

 Design

 Unusual Service Cond.
Including Environment

 Manufacture

 Component Failure

 Installation/Const.

 X Other (specify)

 Operator

Bearing water pressure change.

 Procedure

Cause unknown.

ANALYSIS OF
OCCURRENCE:

Helium circulators, C-2103 and C-2104, were self-turbining when C-2103 tripped with an alarm indication of "loss of bearing water." The standby bearing water pump in the affected loop started which indicates the loop bearing water pressure to the circulators decreased. This eliminates the possibility that the trip signal was caused by the plant protective system. The circulator was restarted and no further problems were encountered.

A check of the latest bearing water switch calibrations did not indicate any reason why C-2103 would trip and C-2104 would not.

The trip is considered spurious and could not be caused to repeat. No explanation for this occurrence could be found.

CORRECTIVE
ACTION:

The circulator was restarted and continued to self-turbine. The other circulator in the loop was operated up to near 3,000 RPM and continued to function normally. No further corrective action is planned at this time.

FAILURE DATA/SIMILAR REPORTED OCCURRENCES:

Unusual Event Report No. 50-267/75/16, dated October 17, 1975, describes the same circulator tripping due to "high pressure separator level". No reason for this trip could be found.

PROGRAMMATIC IMPACT:

None

CODE IMPACT:

None

Submitted by: H. W. Hilkyard, Jr.
H. W. Hilkyard, Jr.
Technical Services Supervisor

Reviewed by: H. Larry Brey by O. Rodgers
H. Larry Brey
Superintendent, Operations

Approved by: Frederic E. Swart by Smith
Frederic E. Swart
Superintendent, Nuclear Production