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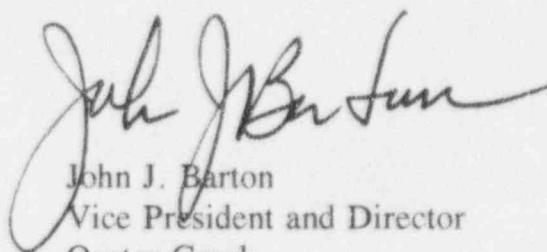
U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555

Dear Sir:

Subject: Oyster Creek Nuclear Generating Station
Docket No. 50-219
Licensee Event Report 95-003

Enclosed is the Licensee Event Report 94-003. This event did not affect the health or safety of the public.

If there are any questions please contact Mr. John Rogers at 609.971.4893.



John J. Barton
Vice President and Director
Oyster Creek

JJB/JJR
Attachment

cc: Administrator, Region I
Senior Resident Inspector
Oyster Creek NRC Project Manager

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LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Oyster Creek, Unit 1	DOCKET NUMBER (2) 05000219	PAGE (3) 1 OF 3
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TITLE (4) Technical Specification Required Surveillance Missed due to Insufficient Administrative Controls

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
05	31	95	95	-- 003 --	0	06	21	95	FACILITY NAME	DOCKET NUMBER 05000

OPERATING MODE (9) N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)									
POWER LEVEL (10) 100	20.402(b)			20.405(c)			50.73(a)(2)(iv)			73.71(b)
	20.405(a)(1)(i)			50.36(c)(1)			50.73(a)(2)(v)			73.71(c)
	20.405(a)(1)(ii)			50.36(c)(2)			50.73(a)(2)(vii)			OTHER
	20.405(a)(1)(iii)			X 50.73(a)(2)(i)			50.73(a)(2)(viii)(A)			(Specify in Abstract below and in Text, NRC Form 366A)
	20.405(a)(1)(iv)			50.73(a)(2)(ii)			50.73(a)(2)(viii)(B)			
	20.405(a)(1)(v)			50.73(a)(2)(iii)			50.73(a)(2)(x)			

LICENSEE CONTACT FOR THIS LER (12)										
NAME Mark Budaj								TELEPHONE NUMBER (Include Area Code) 609.971.4788		

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)										
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)					EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE).		X	NO					

ABSTRACT (16) (Limit to 1400 spaces, i.e., approximately 15 single-spaced lines)

A review of surveillance records conducted on May 31, 1995 revealed that the six month drywell airlock leak rate test which was due on August 17, 1993, was not conducted until September 29, 1993. The delay was attributed to inappropriately applying the 25 % surveillance interval extension which cannot be added to containment pressure testing. The root cause of the inappropriate application of the interval extension was determined to be insufficient administrative controls to ensure compliance with the Technical Specifications.

The safety significance of the delayed surveillance was determined to be minimal as the surveillance which was performed on September 29, 1994 verified that the leak rate of the airlock penetration was acceptable. Corrective action was taken to specifically state that the 25 % extension which may be applied to other surveillances does not apply to the airlock leak rate test.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED BY OMB NO. 3150-0104
EXPIRES 5/31/95

FACILITY NAME (1)		DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
Oyster Creek, Unit 1		05000219	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	2 OF 3
			95	-- 003 --	0	

DATE OF DISCOVERY

The missed surveillance was discovered on May 31, 1995 during a review of completed surveillances.

IDENTIFICATION OF OCCURRENCE

The required leak rate surveillance on the drywell airlock penetration was not performed by its required due date. This is reportable under 10 CFR 50.73(a)(2)(i).

CONDITIONS PRIOR TO DISCOVERY

The plant was at normal temperature and pressure for full power operations in the RUN mode both at the time of discovery and during the period when the test was overdue.

DESCRIPTION OF OCCURRENCE

Technical Specification 4.5.E.5.a was written to address the requirements of 10 CFR 50 Appendix J and states in part:

"The airlock must be tested at least once each 6 month interval..."

Technical Specification 1.24 states in part:

"Each surveillance requirement shall be performed within the specified time interval with a maximum allowable extension not to exceed 25% of the surveillance interval.*"

However, the footnote to Technical Specification 1.24 states:

"* Not applicable to containment leak rate test."

The surveillance was due on August 17, 1993 but was not performed until September 29, 1993. The surveillance was performed within the inappropriate 25% extension, but not within the required six month interval.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION
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APPARENT CAUSE OF OCCURRENCE

Although the surveillance was performed within the inappropriate 25% extension, it was not performed within the required six month surveillance interval. The root cause of this event was insufficient administrative controls to ensure compliance with the six month surveillance interval.

ANALYSIS OF OCCURRENCE AND SAFETY ASSESSMENT

10 CFR 50 Appendix J provides for periodic verification by test of the leak-tight integrity of the reactor primary containment. These requirements have been included in the Oyster Creek Technical Specifications in Section 3.5/4.5. The purpose of this testing is two fold. First, the leak rate tests ensure that leakage through the primary containment, and systems and components penetrating the primary containment, do not exceed allowable leakage rate values as specified in the Technical Specifications. Second, the leak rate tests ensure that periodic surveillance of reactor containment penetrations and isolation valves is performed to ensure that proper maintenance and repairs can be made during the service life of the containment.

This event has been determined to have minimal safety significance as the leak rate surveillance performed on September 29, 1993 verified that the containment airlock penetration was fully operable and would have performed as designed if required.

CORRECTIVE ACTIONS

Upon discovery of the scheduling error, the surveillance task for performing the drywell airlock leak rate test was revised to specifically note that the test must be performed by the specified due date and that the 25% interval extension does not apply.

SIMILAR EVENTS

Voluntary Report 88-023: "Drywell Airlock Not Tested in Accordance with Appendix J"