

Public Service Company of Colorado

P. O. Box 361, Platteville, Colorado 80651

May 7, 1975

MAY 27, 1975

Mr. E. Morris Howard, Director
Nuclear Regulatory Commission
Region IV
Office of Inspection and Enforcement
611 Ryan Plaza Drive
Suite 1000
Arlington, Texas 76012

REF: Facility Operating License
No. DPR-34

Docket No. 50-267

Dear Mr. Howard:

Enclosed please find a copy of Unusual Event Report No. 50-267/75/7, preliminary, and Unusual Event Report No. 50-267/75/8, submitted per the requirements of the Technical Specifications.

Very truly yours,

H. Larry Brey

H. Larry Brey
Superintendent-Operations
Fort St. Vrain Nuclear
Generating Station

HLB:il

cc: Mr. Angelo Giambusso

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inquiry

COPY SENT REGION IV

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REPORT DATE: May 2, 1975

UNUSUAL EVENT

OCCURRENCE DATE: April 28, 1975

FORT ST. VRAIN NUCLEAR GENERATING STATION
PUBLIC SERVICE COMPANY OF COLORADO
P. O. BOX 361
PLATTEVILLE, COLORADO 80651

REPORT NO. 50-267/75/7

IDENTIFICATION OF
OCCURRENCE:

Temporary cables from the auxiliary electrical equipment cabinets to the temporary instrumentation and test equipment were not segregated from the permanent cables according to specification 1-N-2. Color coding on the temporary cables was not consistent with the scheme used in the permanent installation.

CONDITIONS PRIOR
TO OCCURRENCE:

<u> </u>	Steady State Power	<u> </u>	Routine Shutdown
<u> </u>	Hot Shutdown	<u> </u>	Routine Load Change
<u> </u>	Cold Shutdown	<u> </u>	Other (specify)
<u> </u>	Refueling Shutdown	<u> </u>	
<u> X </u>	Routine Startup	<u> </u>	

The major plant parameters at the time of the event were as follows:

Power	Rtr.	<u>16.47</u>	MWth
	Elect.	<u>0</u>	MWe
Secondary Coolant	Pressure	<u>1216</u>	psig
	Temperature	<u>306</u>	°F
	Flow	<u>1.209×10^6</u>	#/hr.
Primary Coolant	Pressure	<u>411</u>	psig
	Temperature	<u>270</u>	°F Core Inlet
		<u>350</u>	°F Core Outlet
	Flow	<u>0.31×10^6</u>	#/hr.

DESCRIPTION OF
OCCURRENCE:

Inspection revealed that some of the temporary cables to the temporary instrumentation and test equipment in the auxiliary electrical equipment room were not segregated from the permanent cables above the auxiliary electrical equipment cabinets, either by physical separation or by fire resistant zipper tubing according to Specification 1-N-2.

APPARENT CAUSE
OF OCCURRENCE:

_____	Design	_____	Unusual Service Cond. Including Environ.
_____	Manufacture	_____	Component Failure
<u> X </u>	Installation/Const.	_____	Other (specify)
_____	Operator	_____	_____
_____	Procedure	_____	_____

ANALYSIS OF
OCCURRENCE:

Adequate separation and segregation should exist between the temporary and permanent cables, according to Specification 1-N-2.

CORRECTIVE
ACTION:

General Atomic Work Authorization No. 99F1849 has been issued to correct color coding on the temporary cables and to zipper tube cables that do not meet segregation criteria according to Specification 1-N-2. The work will be inspected after the work authorization has been completed.

FAILURE DATA/
SIMILAR REPORTED OCCURRENCES:

No failures have occurred at Fort St. Vrain due to electrical segregation and separation. Similar occurrences were reported in Unusual Event No. 50-267/74/2 and 50-267/75/1.

PROGRAMMATIC IMPACT:

None

CODE IMPACT:

None

Submitted:

James H. Reader
James H. Reader
Site Engineer
Fort St. Vrain Nuclear
Generating Station

Approved:

Frederic E. Swart
Frederic E. Swart
Superintendent Nuclear Production
Public Service Company
of Colorado

REPORT DATE: May 2, 1975

UNUSUAL EVENT

OCCURRENCE DATE: April 28, 1975

FORT ST. VRAIN NUCLEAR GENERATING STATION
PUBLIC SERVICE COMPANY OF COLORADO
P. O. BOX 361
PLATTEVILLE, COLORADO 80651

REPORT NO. 50-267/75/8

Preliminary

IDENTIFICATION OF
OCCURRENCE:

Adequate fire stops in the vertical cableways between floors in the reactor building, in the horizontal cable trays and conduits from the auxiliary electrical equipment room in the vertical cable ducts above the stand-by generator rooms were not installed, per drawings issued and per FSAR, Appendix C.3-1.

CONDITIONS PRIOR
TO OCCURRENCE:

<u> </u>	Steady State Power	<u> </u>	Routine Shutdown
<u> </u>	Hot Shutdown	<u> </u>	Routine Load Change
<u> </u>	Cold Shutdown	<u> </u>	Other (specify)
<u> </u>	Refueling Shutdown	<u> </u>	
<u> X </u>	Routine Startup	<u> </u>	

The major plant parameters at the time of the event were as follows:

Power	Rtr.	<u>16.47</u>	MWth
	Elect.	<u>0</u>	MWe
Secondary Coolant	Pressure	<u>1216</u>	psig
	Temperature	<u>306</u>	°F
	Flow	<u>1.209×10^6</u>	#/hr.
Primary Coolant	Pressure	<u>411</u>	psig
	Temperature	<u>270</u>	°F Core Inlet
		<u>350</u>	°F Core Outlet
	Flow	<u>$.31 \times 10^6$</u>	#/hr.

DESCRIPTION OF
OCCURRENCE:

Inspection revealed that adequate fire stops in the vertical cableways between floors in the reactor building, in the horizontal cable trays, and conduits from the auxiliary electrical equipment room and in the vertical cable ducts above the stand-by generator rooms were not installed per drawings issued.

APPARENT CAUSE
OF OCCURRENCE:

[illegible]ANALYSIS OF
OCCURRENCE:

Adequate fire stops should be installed to prevent the spread of fire and smoke and minimize the potential effects of such events to safety.

CORRECTIVE ACTION:

General Atomic Work Authorization No. 92F1852 has been issued to install fire stops as per details on drawings issued and installation will be inspected.

FAILURE DATA/
SIMILAR REPORTED OCCURRENCES:

None

PROGRAMMATIC IMPACT:

None

CODE IMPACT:

Nene

Submitted: *James H. Reider*
James H. Reider
Site Engineer
Fort St. Vrain Nuclear
Generating Station

Approved: Frederic E. Swart
Superintendent Nuclear Production
Public Service Company
of Colorado