Public Service

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Company of Colorado

October 10, 1991
Fort St. Vrain
Unit No. 1
P-91299

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

ATTN: Dr. Seymour H. Weiss, Director
Non-Power Reactor, Decommissioning and
Environmental Project Directorate

Docket No. 50-267

SUBJECT: **BRITISH GRAPHITE TEST REPORT**

Ref: NRC letter, Erickson to Crawford, dated August 30, 1991 (G-91178)

Dear Dr. Weiss:

The purpose of this letter is to forward the British test report that includes evaluation of leaching of radioactivity from graphite. The following is provided as an attachment to this letter:

"Assessment of Management Modes for Graphite from Reactor Decommissioning", Commission of the European Communities, 1984.

This British test report served as the basis of decommissioning assumptions used in determining the tritium leach rate from the Fort St. Vrain graphite, associated with flooding the PCRV for shielding. In particular, your attention is directed to the following sections of the test report that contain applicable information:

Section 2 - discusses material (elemental) composition of the graphite test samples and radioactivity inventories.
Section 6 - discusses the leach rate test methodology and test results.

PSC plans to discuss the British data and use of this information as a basis to estimate the leach rate for tritium contained within graphite at Fort St. Vrain in

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the upcoming PSC/NRC meeting on October 22, 1991, and will provide additional evaluation in PSC's response to the above referenced letter.

If you have any questions concerning this submittal, please contact Mr. M. H. Holmes at (303) 480-6960.

Very truly yours,

Jamine Brey

H.L. Brey
Manager, Nuclear Licensing and
Resource Management

HLB/CRB:cb

Attachment

cc: Regional Administrator, Region IV

Mr. J.B. Baird
Senior Resident Inspector
Fort St. Vrain

Mr. Robert M. Quillin, Director
Radiation Control Division
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