TENNESSEE VALLEY AUTHORITY
CHATTANOOGA, TENNESSEE 37401

January 20, 1976

Dr. Donald F. Knuth, Director
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Dear Dr. Knuth:

BROWNS FERRY NUCLEAR PLANT UNIT 3 - REPORTABLE DEFICIENCY -- POSSIBILITY OF MARGINALLY ADEQUATE SUPPORT STRUCTURE ON MAIN STEAM AND RELIEF VALVE DISCHARGE PIPING

Initial report of the subject reportable deficiency was made to G. R. Klinglar, NRC-IE, Region II, on December 23, 1975. In compliance with paragraph 50.55(e) of 10 CFR Part 50, we submit the following interim report of the subject deficiency.

The possible existance of marginally adequate support structures on the relief valve discharge piping is now under investigation. Teledyne Materials Research is performing and supplying TVA an analysis on each main steam and discharge pipe and its related structures. Upon our receipt and examination of Teledyne's entire analysis of all piping and structures, we will provide you with the required final report.

Very truly yours,

J. E. Gilleland
Assistant Manager of Power

CC: Mr. Norman C. Moseley, Director
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Region II - Suite 813
230 Peachtree Street, NW.
Atlanta, Georgia 30303

831 Power Building
TENNESSEE VALLEY AUTHORITY
CHATTANOOGA, TENNESSEE 37401

January 20, 1976

Dr. Donald F. Knuth, Director
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Dear Dr. Knuth:

BROWNS FERRY NUCLEAR PLANT UNIT 3 - REPORTABLE DEFICIENCY - POSSIBILITY OF MARGINALLY ADEQUATE SUPPORT STRUCTURE ON MAIN STEAM AND RELIEF VALVE DISCHARGE PIPING

Initial report of the subject reportable deficiency was made to G. R. Klingler, NRC-IE, Region II, on December 23, 1975. In compliance with paragraph 50.55(e) of 10 CFR Part 50, we submit the following interim report of the subject deficiency.

The possible existence of marginally adequate support structures on the relief valve discharge piping is now under investigation. Teledyne Materials Research is performing and supplying TVA an analysis on each main steam and discharge pipe and its related structures. Upon our receipt and examination of Teledyne's entire analysis of all piping and structures, we will provide you with the required final report.

Very truly yours,

J. E. Gilleland

J. E. Gilleland
Assistant Manager of Power

CC: Mr. Norman C. Moseley, Director
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Region II - Suite 818
230 Peachtree Street, NW.
Atlanta, Georgia 30303

60069

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

June 10, 1976

Note to K. Seyfrit

PROPOSED TRANSFER OF LEAD RESPONSIBILITY, SERIAL NO. IE-RID-76-08

As discussed in our telephone conversation today, the questions requested by the subject proposed transfer of lead responsibility will be considered and resolved as part of our currently ongoing generic activities on Mark I containment problems. We therefore do not think it necessary or appropriate to complete this transfer of lead responsibility.

Karl R. Goller

Karl R. Goller, Assistant Director
for Operating Reactors
Division of Operating Reactors

cc: J. Guibert
L. Shao

Dupe

8309170109

Date:

Serial No.: IE-RID-76-08

TRANSFER OF LEAD RESPONSIBILITY

To: K. R. Goller, Assistant Director for Operating Reactors, DOR

Subject: ADEQUACY OF SUPPORTS FOR SAFETY/RELIEF VALVE PIPING SYSTEMS AT BROWNS FERRY

Responsible Branch Chief: K. V.Seyfrit

Description of Item Requiring Resolution:

On December 23, 1975, in accordance with 10 CFR 50.55(e), TVA reported the possible existence of marginally adequate supporting systems for the main steam and relief valve piping located in the drywell of Browns Ferry Unit 3. The uncertain status of the adequacy of these piping system supports was revealed as a result of plans to replace two Target Rock safety/relief valves with Crosby valves on the Unit 3 facility. This change of valves led to a reanalysis of the stress loadings on these piping systems and the discovery of possibly inadequate supports for the piping systems of Units 1 and 2 with the present Target Rock valve installation.

This problem appears to have generic implications to BWRs currently in operation. BWRs in operation prior to 1972 were reassessed as to adequacy of these piping systems following the events at Turkey Point and Surry. The recent development at Browns Ferry, raises questions as to the capability of piping supports for the relief valve discharge piping in the drywell to withstand the loads resulting from safety/relief valve operation over the design lifetime of the plant.

Recommendations and Proposed Course of Action:

1. The Office of Nuclear Reactor Regulation (NRR) will review and evaluate the adequacy of the modified piping support systems being installed at Browns Ferry Units 1, 2 and 3.

2. The NRR will determine the adequacy of these support systems at other operating BWRs and establish the acceptance criteria for these piping systems.

3. At the completion of the review and evaluation NRR will inform Office of Inspection and Enforcement of the results and conclusions which they have developed. If at the completion of the review and evaluation there is a need for additional information from or new requirements for the Licensee, both the Licensee and IE will be advised by NRR.

Dupe

~~8308190112~~

4. IE will provide information related to this subject as requested and will conduct inspections of any required changes to piping support systems to assure conformance to license, code or other imposed requirements.

Reference: Memo, F. J. Long, IE:II to K. V. Seyfrit, w/encl., dd. 3/3/76

Concurrence:

Boyce H. Grier 3/31/76

B. H. Grier, Director, Division of Reactor Inspection Programs, IE / Date

K. R. Goller, Assistant Director for Operating Reactors, DOR Date

cc: E. Volgenau, IE
R. S. Boyd, DPM
J. G. Davis, IE
R. A. Purple, DOR
T. V. Wambach, DOR
K. V. Seyfrit, IE
J. H. Sniezek, IE
G. W. Reinmuth, IE
C. J. DeBevec, IE
D. Thompson, IE
G. Roy, IE
G. Gower, IE
J. P. O'Reilly, IE:I
N. C. Moseley, IE:II
W. C. Seidle, IE:II
C. E. Murphy, IE:II
J. G. Keppler, IE:III
E. M. Howard, IE:IV
R. H. Engelken, IE:V
T. Abell, MIPC
H. K. Shapar, ELD

UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
230 PEACHTREE STREET, N. W. SUITE 818
ATLANTA, GEORGIA 30303

March 3, 1976

Karl V. Seyfrit, Office of Inspection and Enforcement, Headquarters
THRU: N. C. Moseley, Director, Office of Inspection and Enforcement, Region II

TVA, BROWNS FERRY UNITS 1, 2, AND 3, MARGINALLY ADEQUATE SUPPORT STRUCTURES ON RELIEF VALVE DISCHARGE PIPING

TVA reported the possibility of marginally adequate support structures on main steam and relief valve piping under 50.55(e) requirements December 23, 1975. An interim report received January 22, 1976, stated that a final report would be sent upon TVA's receipt and analysis of all piping and structures provided by Teledyne. A telecon check with TVA has indicated that the final report will be issued mid-April.

An inspection was performed by the IE:II Construction Branch the week of February 23, 1976, to determine the design and construction status of modifications to the relief valve discharge piping. It appears that the support structure problem could be generic to BWR sites. It is therefore requested that the Lead Responsibility for this problem be transferred to Licensing to determine the adequacy of the new supports added at the Browns Ferry site and to determine the generic implications for other BWR sites.

F. J. Long, Chief
Reactor Operations and Nuclear
Support Branch

cc: G. Gower
G. Roy
W. C. Seidle
C. E. Murphy

8306170117

TENNESSEE VALLEY AUTHORITY
CHATTANOOGA, TENNESSEE 37401

January 20, 1976

Dr. Donald F. Knuth, Director
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Dear Dr. Knuth:

BROWNS FERRY NUCLEAR PLANT UNIT 3 - REPORTABLE DEFICIENCY -- POSSIBILITY OF MARGINALLY ADEQUATE SUPPORT STRUCTURE ON MAIN STEAM AND RELIEF VALVE DISCHARGE PIPING

Initial report of the subject reportable deficiency was made to G. R. Klinglar, NRC-IE, Region II, on December 23, 1975. In compliance with paragraph 50.55(e) of 10 CFR Part 50, we submit the following interim report of the subject deficiency.

The possible existance of marginally adequate support structures on the relief valve discharge piping is now under investigation. Teledyne Materials Research is performing and supplying TVA an analysis on each main steam and discharge pipe and its related structures. Upon our receipt and examination of Teledyne's entire analysis of all piping and structures, we will provide you with the required final report.

Very truly yours,

J. E. Gilleland
Assistant Manager of Power

CC: Mr. Norman C. Moseley, Director
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Region II - Suite 813
230 Peachtree Street, NW.
Atlanta, Georgia 30303

Dupe 8308190119