

ENCLOSURE 1

PROPOSED TECHNICAL SPECIFICATION REVISIONS
TVA BFNP TS 176 SUPPLEMENT 8
BROWNS FERRY NUCLEAR PLANT
UNITS 1, 2, AND 3

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TABLE 3.7.A
PRIMARY CONTAINMENT ISOLATION VALVES

Group	Valve Identification	Number of Power Operated Valves		Maximum Operating Time (sec.)	Normal Position	Action on Initiating Signal
		Inboard	Outboard			
1	Main steamline isolation valves (FCV-1-14, 26, 37, 65; 1-15, 27, 38, & 52)	4	4	3 < T < 5	O	GC
1	Main steamline drain isolation valves FCV-1-55 & 1-56	1	1	15	O	GC
1 *	Reactor Water sample line isolation valves	1	1	5	C	SC
2	RHRS shutdown cooling supply isolation valves FCV-74-48 & 47	1	1	40	C	SC
2	RHRS - LPCI to reactor FCV-74-53, 67		2	30	C	SC
2	Reactor vessel head spray isolation valves FCV-74-77, 78	1	1	30	C	SC
2	RHRS flush and drain vent to suppression chamber FCV-74-102, 103, 119, & 120		4	20	C	SC
2	Suppression Chamber Drain FCV 75-57, -58		2	15	O(1)	GC
2	Drywell equipment drain discharge isolation valves FCV-77-15A, & 15B		2	15	O	GC
2	Drywell floor drain discharge isolation valves FCV-77-2A & 2B		2	15	O	GC

*These valves isolate only on reactor vessel low low water level (470") and main steam line high radiation of Group 1 isolations.

NOTES

1. These valves are normally open when the pressure suppression head tank is aligned to serve the RHR and CS discharge piping and closed when the condensate head tank is used to serve the RHR and CS discharge piping. (See Specification 3.5.H)

TABLE 3.7.A
PRIMARY CONTAINMENT ISOLATION VALVES

Group	Valve Identification	Number of Power Operated Valves		Maximum Operating Time (sec.)	Normal Position	Action on Initiating Signal
		Inboard	Outboard			
1	Main steamline isolation valves (PCV-1-14, 26, 37, & 51; 1-15, 27, 38 & 52)	4	4	3 < T < 5	0	GC
1	Main steamline drain isolation valves (PCV-1-55 & 1-56)	1	1	15	0	GC
1*	Reactor Water sample line isolation valves	1	1	5	C	SC
2	RHRS shutdown cooling supply isolation valves (PCV-74-48 & 47)	1	1	40	C	SC
2	RHRS - LPCI to reactor (PCV-74-53 & 67)		2	30	C	SC
2	Reactor vessel head spray isolation valves (PCV-74-77 & 78)	1	1	30	C	SC
2	RHRS flush and drain vent to suppression chamber (PCV-74-102, 103, 119, & 120)		4	20	C	SC
2	Suppression Chamber Drain (PCV-75-57 & 58)		2	15	0(1)	GC
2	Drywell equipment drain discharge isolation valves (PCV-77-15A & 15B)		2	15	0	GC
2	Drywell floor drain discharge isolation valves (PCV-77-2A & 2B)		2	15	0	GC

Notes:

1. These valves are normally open when the pressure suppression head tank is aligned to serve the RHR and CS discharge piping and closed when the condensate head tank is used to serve the RHR and CS discharge piping. (See specification 3.5.11)

*These valves isolate only on reactor vessel low low water level, (470") and main steam line high radiation of Group 1 isolations.

TABLE 3.7.A
PRIMARY CONTAINMENT ISOLATION VALVES

Group	Valve Identification	Number of Power Operated Valves		Maximum Operating Time (sec.)	Normal Position	Action on Initiating Signal
		Inboard	Outboard			
1	Main steamline isolation valves (PCV-1-14, 26, 37, & 51; 1-15, 27, 38 & 52)	4	4	1 < T < 5	0	GC
1	Main steamline drain isolation valves (PCV-1-55 & 1-56)	1	1	15	0	GC
1 *	Reactor water sample line isolation valves	1	1	5	C	SC
2	RHRS shutdown cooling supply isolation valves (PCV-74-48 & 47)	1	1	40	C	SC
2	RHRS - LPCI to reactor (PCV-74-53 & 67)		2	30	C	SC
2	Reactor vessel head spray isolation valves (PCV-74-77 & 78)	1	1	30	C	SC
2	RHRS flush and drain vent to suppression chamber (PCV-74-102, 103, 119, & 120)		4	20	C	SC
2	Suppression Chamber Drain (PCV-75-57 & 58)		2	15	0(1)	GC
2	Drywell equipment drain discharge isolation valves (PCV-77-15A & 15B)		2	15	0	GC
2	Drywell floor drain discharge isolation valves (PCV-77-2A & 2B)		2	15	0	GC

*These valves isolate only on reactor vessel low low water level (470") and main steam line high radiation of Group 1 isolations.

NOTES

1. These valves are normally open when the pressure suppression head tank is aligned to serve the RHR and CS discharge piping and closed when the condensate head tank is used to serve the RHR and CS discharge piping. (See Specification 3.5.H)

ENCLOSURE 2
DESCRIPTION AND JUSTIFICATION
TVA BFP TS 176 SUPPLEMENT 8

Description

Page 250 (Units 1 and 2)

Page 262 (Unit 3)

For valves FCV 75-57, 75-58, Suppression Chamber drain, it is proposed to change the Normal Position from "closed" to "open" and Action on Initiation Signal from "stays closed" to "goes closed." Also, it is proposed to add the note: "These valves are normally open when the pressure suppression head tank is aligned to serve the RHR and CS discharge piping and closed when the condensate head tank is used to serve the RHR and CS discharge piping. (See Specification 3.5.H)"

Justification

These changes are proposed for clarification of the normal position of the suppression chamber drain valves FCV 75-57 and 75-58. The changes also correct typographical errors in the designation of the suppression chamber drain valves to reflect proper valve numbers and proper action of valves on initiating signal.

These changes reflect the correct valve positions for operating the pressure suppression chamber head tank system. It will not be possible to operate the pressure suppression chamber head tank system with the valve alignment shown in the existing technical specifications. This system was installed and will be used primarily to alleviate certain NRC concerns.

Page 250 for unit 1 includes the changes proposed in TS 176 Supplement 2. Description and justification for those changes was provided in that submittal.

Page 262 for unit 3 includes the change proposed in TS 176 Supplement 1, the addition of the footnote on the reactor water sample line isolation valves. As explained fully in that submittal the footnote was added to more accurately represent the isolation trips for these valves.

Safety Analysis

This revision is being made to clarify the normal position of the suppression chamber drain valves FCV 75-57 and 75-58. It does not alter in any way the present method of operation or the overall safety of the plant. These valves will still go to the safe position in the event of a containment isolation signal.

ENCLOSURE 3
BROWNS FERRY NUCLEAR PLANT
SIGNIFICANT HAZARDS CONSIDERATION
FOR

PROPOSED TECHNICAL SPECIFICATION CHANGES

1. Does the proposed amendment involve a significant increase in the probability or consequences of an accident previously evaluated?

No - This revision does not alter in any way the system operation as described in the FSAR.

2. Does the proposed amendment create the probability of a new or different kind of accident from any accident previously evaluated?

No - This revision does not alter in any way the present method of operation or the overall safety of the plant.

3. Does the proposed amendment involve a significant reduction in a margin of safety?

No- This revision does not in any way alter a proscribed margin of safety.