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ComEd

June 7, 1995
BW/95-0064

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Subject: Braidwood Station Unit 1
Steam Generator Interim Plugging Criteria 90 Day Report
NPF-72; NRC Docket No. 50-456

- References:
1. November 23, 1994, Letter from Denise M. Saccomando to Office of Nuclear Reactor Regulations, Braidwood Unit 1 Mid-Cycle Inspection Plans.
 2. February 27, 1995, Letter from Denise M. Saccomando to U.S. Nuclear Regulatory Commission, Additional Information Regarding Braidwood Unit 1 Mid-Cycle Outage Plan.
 3. March 1, 1995, Letter from Denise M. Saccomando to U.S. Nuclear Regulatory Commission, Braidwood Station Unit 1 Mid-Cycle Inspection.
 4. March 8, 1995, Letter from Denise M. Saccomando to U.S. Nuclear Regulatory Commission, Safety Assessment for Braidwood Unit 1 Mid-Cycle Outage A1M05.
 5. Draft Generic letter, "Voltage-Based Repair Criteria for the Repair of Westinghouse Steam Generator Tubes Affected by Outside Diameter Stress Corrosion Cracking," issued for public comment on August 12, 1994.

In Reference 1, Braidwood Station committed to follow the requirements of Reference 5 with exceptions as noted in references 1 through 3. One of the requirements of the Draft Generic letter is to issue the final results of the inspection and tube integrity evaluation to the staff within 90 days following restart.

Pursuant to this requirement, ComEd is submitting the enclosed report of results from the Braidwood Unit 1 End-of-Mid-Cycle 5 (A1M05) steam generator inspection. Startup (Mode 2) from A1M05 was conducted on March 13, 1995.

Attachment A to this letter contains the 90 day report for Braidwood Unit 1. The following information is included in the report.

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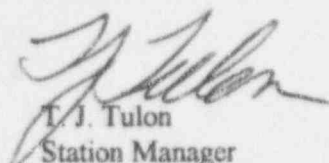
1. A detailed summary of the AIM05 inspection results is provided in tabular and graphical form through each of the following:
 - a. The results of the tube integrity evaluation, including the conditional burst probability and main steam line break leak rate (preliminary results were provided in Reference 4 prior to resumption of power operation from AIM05).
 - b. An updated evaluation of the voltage growth rates.
2. The estimated leakage based on actual measured EOMC voltage distribution did not exceed the leak limit.
3. No circumferential crack-like indications were detected at the tube support plate intersections; although, circumferential crack-like indications were detected in the tube roll transition region at the top of the tubesheet.
4. No indications were identified to extend beyond the confines of the tube support plate.
5. The calculated conditional burst probability did not exceed 1×10^{-2} .

No tubes were removed from the Steam Generators; therefore, no metallurgical results of pulled tubes are available.

Attachment B contains a review of the programs which were implemented to mitigate SG tube degradation. This review was performed to assess the effectiveness of each program on the increase in voltage growth rates.

Please direct any questions regarding this submittal to Doug Huston, Braidwood Licensing Supervisor, (815) 458-2801, extension 2511.

Yours truly,


T. J. Tulon
Station Manager
Braidwood Nuclear Station

TJT/DH/dla

Attachments

cc: S. G. DuPont, Sr. Resident Inspector, Braidwood
R. R. Assa, Braidwood Project Manager, NRR
J. B. Martin, Regional Administrator, RIII

Attachment A

Braidwood Unit 1

1995 Interim Plugging Criteria 90 Day Report