

LICENSEE EVENT REPORT

CONTROL BLOCK:

(PLEASE PRINT ALL REQUIRED INFORMATION)

LICENSEE NAME		LICENSE NUMBER				LICENSE TYPE				EVENT TYPE								
01	I	L	Q	A	0	2	0	0	0	0	0	4	1	1	1	1	0	1
7	8	9	14	15	25	26	30	31	32									

CATEGORY		REPORT TYPE		REPORT SOURCE		DOCKET NUMBER				EVENT DATE				REPORT DATE					
01	CON'T	P	O	7	L	0	5	0	1	0	2	6	5	1	0	1	4	7	5
7	8	57	58	59	60	61	68	69	74	75	80								

EVENT DESCRIPTION

02	WITH UNIT 2 IN THE COLD SHUTDOWN CONDITION																		80
03	WELD EXAMINATIONS INDICATED A CRACK ON																		80
04	THE B LOOP OF THE RX RECIRC DISCHARGE																		80
05	BYPASS LINE. BECAUSE REPETITIVE CRACKS HAVE BEEN																		80
06	FOUND, BOTH BYPASS LINES WILL BE REMOVED																		80

SYSTEM CODE		CAUSE CODE		COMPONENT CODE				PRIME COMPONENT SUPPLIER		COMPONENT MANUFACTURER				VIOLATION	
07	C	B	E	X	X	X	X	X	W	X	9	9	9	W	
7	8	9	10	11	12	17	43	44	47	48					

CAUSE DESCRIPTION

08	APPARENT CAUSE IS INTERGRANULAR STRESS																		80
09	ASSISTED CORROSION.																		80
10																			80

FACILITY STATUS		% POWER		OTHER STATUS				METHOD OF DISCOVERY		DISCOVERY DESCRIPTION				
11	G	0	0	0	NA	E	ULTRASONIC TESTING							
7	8	9	10	12	13	44	45	46						

FORM OF ACTIVITY RELEASED		CONTENT OF RELEASE		AMOUNT OF ACTIVITY				LOCATION OF RELEASE			
12	Z	Z	NA				NA				
7	8	9	10	11	44	45					

PERSONNEL EXPOSURES

NUMBER		TYPE		DESCRIPTION				
13	0	0	0	Z	NA			
7	8	9	11	12	13			

PERSONNEL INJURIES

NUMBER		DESCRIPTION			
14	0	0	0	NA	
7	8	9	11	12	

OFFSITE CONSEQUENCES

15	NA																		80
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LOSS OR DAMAGE TO FACILITY

TYPE		DESCRIPTION			
16	Z	NA			
7	8	9	10		

PRIORITY

17	8306170247 751014 PDR ADOCK 05000265 S PDR																		80
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ADDITIONAL FACTORS

18	DESCRIPTION OF EVENT (CONT'D), AS PER																		80
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19	QUAD CITIES MODIFICATION M-4-2-75-27. (AQ-50-265/75-3)																		80
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NJK-75-525

October 24, 1975

Director Of Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

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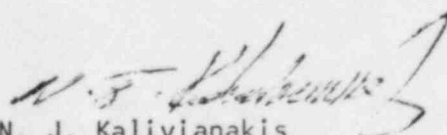
Reference: Quad-Cities Nuclear Power Station
Docket No. 50-265, DPR-30, Unit 2
Appendix A, Sections 1.0.A.5, 6.6.B.1.a

Enclosed please find Abnormal Occurrence Report No. AO 50-265/75-38 for Quad-Cities Nuclear Power Station. This occurrence was previously reported to Region III, Office of Inspection and Enforcement by telephone on October 13, 1975 and to you and Region III, Office of Inspection and Enforcement by telecopy on October 14, 1975.

This report is submitted to you in accordance with the requirements of Technical Specification 6.6.B.1.a.


Very truly yours,

COMMONWEALTH EDISON COMPANY
QUAD-CITIES NUCLEAR POWER STATION


N. J. Kalivianakis
Station Superintendent

NJK/JCV/lk

cc: Region III, Office of Inspection and Enforcement
G. A. Abrell

COPY SENT REGION 

REPORT NUMBER: AO 50-265/75-38

REPORT DATE: October 24, 1975

OCCURRENCE DATE: October 14, 1975

FACILITY: Quad-Cities Nuclear Power Station
Cordova, Illinois 61242

IDENTIFICATION OF OCCURRENCE:

Crack indications in Unit 2 "B" recirculation loop discharge valve bypass line upper weld-onlet.

CONDITIONS PRIOR TO OCCURRENCE:

Unit 2 was in the cold shutdown condition.

DESCRIPTION OF OCCURRENCE:

In accordance with the provisions of NRC R.O. Bulletin 74-10A, ultrasonic testing of accessible welds in the bypass piping lines around the recirculation pump discharge valves was performed during the Unit 2 core maintenance outage.

On October 14, 1975 a crack indication was confirmed on the "B" loop at the junction of the bypass line with the 28 inch discharge header on the downstream side of the bypass valve. The crack indication was in the heat affected zone on the pipe side of the pipe-to-weldolet weld.

DESIGNATION OF APPARENT CAUSE OF OCCURRENCE:

Equipment Failure

The mode of failure and thus the cause of this occurrence has not been established as of the date of this report. The apparent cause is believed to be the same as that which has caused similar cracks in recirculation bypass lines in the past, that is, intergranular stress assisted corrosion.

ANALYSIS OF OCCURRENCE:

Early detection of the crack prevented violation of the leak rate limits set forth in the Technical Specifications and as modified by I.E. Bulletin 74-10B. No radioactivity was released to the environs nor was any health hazard to the public or plant personnel posed. Thus, the safety implications of this occurrence were minimal.

October 24, 1975

CORRECTIVE ACTION:

The existing recirculation pump discharge valve bypass piping on both loops A and B will be removed. The removal will be accomplished by installing 16 inch 304 L low carbon stainless steel spool pieces with pipe end caps. This work is documented by Quad-Cities Modification M-4-2-75-27. An information letter describing the details of the modification will be provided in the near future. This modification has been discussed and reviewed with various Region III NRC representatives.

FAILURE DATA:

On September 16, 1974 Quad-Cities Unit 2 experienced a crack at weld 2BB-F 10 on the B loop. Corrective action at that time was to replace the weld and a short section of pipe.

A second ultrasonic inspection, conducted on Quad-Cities Unit 2 on December 23, 1974 revealed two indications on bypass loop A and one additional indication on bypass loop B. Corrective action at that time was to replace both the A and the B loop recirculation pump discharge valve bypass piping.

On January 10, 1975 an examination of Quad-Cities Unit 1 recirculation pump discharge valve bypass piping revealed a crack on the A loop weldolet running along the 4 inch side of the weld and an indication of a crack on the B loop weldolet running 1/2 inch to 3/4 inch from the weld head. Corrective action was to replace both the A and the B loop recirculation pump discharge valve bypass piping.

Similar cracks were also found at Dresden and Millstone stations.

The pipe that failed in all cases was 304 stainless steel, four inch diameter with a wall thickness of 0.377 inches nominal.