

LICENSEE EVENT REPORT

CONTROL BLOCK: 1 2 3 4 5 6

(PLEASE PRINT ALL REQUIRED INFORMATION)

LICENSEE NAME: 01 I L Q A D Z 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32

REPORT TYPE: 01 CONT 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

EVENT DESCRIPTION

02 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32

SYSTEM CODE: 07 1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32

CAUSE DESCRIPTION

08 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32

11 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32

PERSONNEL EXPOSURES

13 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32

PERSONNEL INJURIES

14 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32

OFFSITE CONSEQUENCES

15 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32

LOSS OR DAMAGE TO FACILITY

16 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32

PUBLICITY

17 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32

ADDITIONAL FACTORS

18 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32

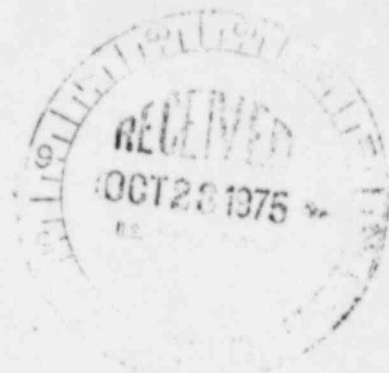
19 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32

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NJK-75-527

October 24, 1975

Director of Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D. C. 20555

Reference: Quad-Cities Nuclear Power Station
Docket No. 50-265, DPR-30, Unit 2
Appendix A, Sections 1.0.A.2, 3.1.A, Table 3.1.1.c, 6.6.1.B.a

Enclosed please find Abnormal Occurrence Report No. A0 50-265/75-40 for Quad-Cities Nuclear Power Station. This occurrence was previously reported to Region III, Office of Inspection and Enforcement by telephone on October 16, 1975 and to you and Region III, Office of Inspection and Enforcement by telecopy on October 16, 1975.

This report is submitted to you in accordance with the requirements of Technical Specification 6.6.B.1.a.

Very truly yours,

COMMONWEALTH EDISON COMPANY
QUAD-CITIES NUCLEAR POWER STATION

N. J. Kalivianakis
Station Superintendent

NJK/MPF/lk

cc: Region III, Office of Inspection and Enforcement
G. A. Abrell

10201
COPY SENT REGION III

REPORT NUMBER: AO 60-265/75-40

REPORT DATE: October 24, 1975

OCCURRENCE DATE: October 16, 1975

FACILITY: Quad-Cities Nuclear Power Station
Cordova, Illinois 61242

IDENTIFICATION OF OCCURRENCE:

High drywell pressure switches PS-2-1001-88 A, B, and D out of calibration.

CONDITIONS PRIOR TO OCCURRENCE:

Unit Two was shutdown for a core maintenance outage with the mode switch in REFUEL.

DESCRIPTION OF OCCURRENCE:

While performing routine surveillance on October 16, 1975 at approximately 1:00 p.m., three of the four high drywell pressure scram switches were found to exceed the Technical Specification setpoint limit of ≤ 2.0 psig. The as-found setpoints were as follows:

PS 2-1001-88A	2.35 psig
PS 2-1001-88B	2.15 psig
PS 2-1001-88D	2.20 psig

DESIGNATION OF APPARENT CAUSE OF OCCURRENCE:

Equipment Failure

The apparent cause of this occurrence is designated as instrument drift. The switch involved is a static-o-ring, Model 12N-AA5-PP. The setpoints were calibrated at 1.95 ± 0.05 psig on September 9, 1975 for PS 2-1001-88A and B and on September 10, 1975 for PS 2-1001-88D.

ANALYSIS OF OCCURRENCE:

Pressure switches PS-2-1001-88 A through D are part of the Reactor Protection System one-out-of-two-twice logic. They provide a scram signal during a loss of coolant accident to minimize the energy which must be accommodated by the emergency core cooling systems and to prevent the reactor from going critical following the accident.

Since PS-2-1001-88A and C are in Channel "A" and PS-2-1001-88 B and D are in Channel "B" the scram would have occurred at a drywell pressure of 2.15 psig rather than the desired 2.00 psig. The effect of this elevated trip

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point on the functions described above is minimal because there are several other scram signals that would actuate prior to the drywell high-pressure signal and thus accomplish the desired scram. The health and safety of the public were not affected by this occurrence.

CORRECTIVE ACTION:

The corrective action involved an immediate recalibration and functional test of the affected pressure switches. This recalibration resulted in the following acceptable setpoints:

2-1001-88A	1.93 psig
2-1001-88B	1.90 psig
2-1001-88D	1.90 psig

FAILURE DATA:

Instrument setpoint drifts of the drywell high pressure sensors have occurred in the past. Through continued surveillance recalibration and the existing setpoint margin for possible drift, occurrences of this nature are expected to be minimal in severity and frequency.