

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)

Joseph M. Farley - Unit 1

DOCKET NUMBER (2)

0 5 0 0 0 3 4 8 1 OF 0 2

PAGE (3)

TITLE (4)

Reactor Protection System Actuation While In Mode 3

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)									
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)							
0	4	2	1	8	4	8	4	0	1	2	0	5	0	0	0			
0	4	2	1	8	4	8	4	0	5	2	1	8	4	0	5	0	0	0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)

OPERATING MODE (9)	20.402(b)	20.405(a)(1)(i)	20.405(a)(1)(ii)	20.405(a)(1)(iii)	20.405(a)(1)(iv)	20.405(a)(1)(v)	20.406(a)	50.36(a)(1)	50.36(a)(2)	50.73(a)(2)(i)	50.73(a)(2)(ii)	50.73(a)(2)(iii)	50.73(a)(2)(iv)	50.73(a)(2)(v)	50.73(a)(2)(vi)	50.73(a)(2)(vii)	50.73(a)(2)(viii)(A)	50.73(a)(2)(viii)(B)	50.73(a)(2)(ix)	73.71(b)	73.71(c)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)	
3																							
POWER LEVEL (10)	0	0	0																				

LICENSEE CONTACT FOR THIS LER (12)

NAME

TELEPHONE NUMBER

W. G. Hairston, III

AREA CODE

2 0 5 8 9 9 - 5 1 5 6

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)	NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
	X				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

At 1133 on 4-21-84, with the unit shut down and response time testing in progress, a B train reactor trip signal, which was not part of the planned sequence, occurred. The reactor trip signal was caused by improper performance of a surveillance test procedure. Health/safety of the public was not affected.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Joseph M. Farley - Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 3 4 8 8 4 - 0 1 2 - 0 0	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 4	0 1 2	0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On 4-21-84, the unit was in Mode 3 with surveillance testing in progress following the Cycle V-VI refueling outage. As part of this program, I & C personnel were performing FNP-1-STP-256.12 (Turbine Trip - Reactor Trip Response Time Test). In accordance with the procedure a B train P-9 signal (reactor power greater than 35%) existed on power range channel N-41. At 1133, other I & C personnel began performing FNP-1-STP-228.6 (Nuclear Instrumentation System Power Channel N-42 Calibration and Functional Test) which involves generating A and B train P-9 signals on power range channel N-42. When this was done a B train reactor trip signal occurred due to a P-9 signal on 2 out of 4 power range channels while the turbine was tripped.

Since only a B train reactor trip signal was generated, only the B train reactor trip breaker opened. The plant operator immediately tripped the A train reactor trip breaker manually. This event had no effect on the unit nor did it cause any control rod movement because the unit was in Mode 3 with all control rods fully inserted.

This event was caused by personnel error. The I & C personnel performing FNP-1-STP-228.6 failed to ensure that no other power range channels were in test prior to commencing work, as required in the initial conditions section of the procedure. The personnel involved have been counseled concerning the necessity of adherence to procedures.

Mailing Address

Alabama Power Company
600 North 18th Street
Post Office Box 2641
Telephone 205 783-6090

R. P. McDonald

Senior Vice President-
Nuclear Generation
Flintridge Building



Alabama Power

the southern electric system

May 21, 1984

Docket No. 348

Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Sir:

Joseph M. Farley Nuclear Plant, Unit 1, Licensee Event Report No. LER 84-012-00 is forwarded in accordance with 10CFR50.73 to provide 30 day written notification of this occurrence. This event was not determined to be reportable until 5-21-84, 30 days after the event. Therefore, the four hour non emergency notification in accordance with 10CFR50.72(b)(2)ii was not performed.

If you have any questions, please advise.

Yours very truly,

R. P. McDonald

RPM/DSM:sam

Enclosure

xc: IE, Region II

IE22
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