

LICENSEE EVENT REPORT

CONTROL BLOCK: 1 2 3 4 5 6

[PLEASE PRINT ALL REQUIRED INFORMATION]

LICENSEE NAME										LICENSE NUMBER										LICENSE TYPE										EVENT TYPE																	
01		I		L		Q		A		D		2		0		0		-		0		0		0		0		-		0		0		4		1		1		1		1		0		3	
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	38	39	40	41	42	43	44	45	46	47	48	49	50				

CATEGORY										REPORT TYPE										REPORT SOURCE										DOCKET NUMBER										EVENT DATE										REPORT DATE									
01		CON'T		57		58		59		60		61		5		0		-		0		2		6		5		0		6		2		8		7		6		0		7		2		7		7		6									
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	38	39	40	41	42	43	44	45	46	47	48	49	50																

EVENT DESCRIPTION

02		At 2:05 am on June 28, 1976 it was noticed that the Unit 2 Refuel Floor Radiation																																																80	
03		Monitor 2B was out of service. Upon further investigation it was discovered that																																																80	
04		it had been out of service greater than the Technical Specification limit of 24 hours.																																																80	
05		The reactor building vent system was then isolated per Specification 3.2.F.2. Refuel																																																80	
06		Floor Monitor 2A, Unit 1 Refuel Floor monitors, and all Reactor Building Vent duct																																																80	

SYSTEM CODE										CAUSE CODE										COMPONENT CODE										PRIME COMPONENT SUPPLIER										COMPONENT MANUFACTURER										VIOLATION									
07		B		A		D		I		N		S		T		R		U		N		G		O		8		0		N																													
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	38	39	40	41	42	43	44	45	46	47	48	49	50																

CAUSE DESCRIPTION

08		(Proximate Cause-Procedure Inadequacy) The identification of the Refuel Floor Radiation																																																80	
09		Monitors was unclear on Outage Report QOS 1700-01 Rev. 1. A procedure revision was																																																80	
10		made to correct this deficiency. This is the first occurrence of this type for the																																																80	

FACILITY STATUS										% POWER										OTHER STATUS										METHOD OF DISCOVERY										DISCOVERY DESCRIPTION									
11		F		0		7		7		NA		E		NA																																			
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	38	39	40	41	42	43	44	45	46	47	48	49	50						

FORM OF ACTIVITY RELEASED										CONTENT OF RELEASE										AMOUNT OF ACTIVITY										LOCATION OF RELEASE													
12		Z		Z		NA		NA		NA																																	
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	38	39	40	41	42	43	44	45	46	47	48	49	50

PERSONNEL EXPOSURES

NUMBER										TYPE										DESCRIPTION																							
13		0		0		0		NA																																			
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	38	39	40	41	42	43	44	45	46	47	48	49	50

PERSONNEL INJURIES

NUMBER										DESCRIPTION																																	
14		0		0		0		NA																																			
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	38	39	40	41	42	43	44	45	46	47	48	49	50

OFFSITE CONSEQUENCES

15		NA																																																80	
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LOSS OR DAMAGE TO FACILITY

TYPE										DESCRIPTION																																	
16		Z		NA																																							
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	38	39	40	41	42	43	44	45	46	47	48	49	50

PUBLICITY

17		NA		8306160697 760727 PDR ADOCK 05000265 S PDR																																														80	
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ADDITIONAL FACTORS

18		(EVENT DESCR. CONT.) radiation monitors were operable and capable of performing their																																																80	
19		adequate to prevent recurrence. (RO 50-265/76-8)																																																80	

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U. S. Nuclear Regulatory Commission

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NJK-76-280

July 28, 1976

J. Keppler, Regional Director
Office of Inspection and Enforcement
Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137

Reference: Quad-Cities Nuclear Power Station
Docket No. 50-265, DPR-30, Unit 2
Appendix A, Sections 3.2.F.2, 6.6.B.2.c

Enclosed please find Reportable Occurrence Report No. RO 50-265/76-8 for Quad-Cities Nuclear Power Station.

This report is submitted to you in accordance with the requirements of Technical Specification 6.6.B.2.

Very truly yours,

COMMONWEALTH EDISON COMPANY
QUAD-CITIES NUCLEAR POWER STATION

N. J. Kalivianakis
N. J. Kalivianakis
Station Superintendent

COPY SENT REGION III

NJK/LLH/lk

cc: G. A. Abreil

JUL 30 1976