

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Sequoyah, Unit 1										DOCKET NUMBER (2) 0 5 0 0 0 3 2 7				PAGE (3) 1 OF 0 2										
TITLE (4) Reactor Trip																								
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)														
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)											
0	4	1	7	8	4	8	4	0	2	6	0	0	0	5	1	5	8	4	0	5	0	0	0	0
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)																						
POWER LEVEL (10)		20.402(b)				20.406(e)				50.73(a)(2)(iv)				73.71(b)										
0		20.406(a)(1)(i)				50.38(e)(1)				50.73(a)(2)(v)				73.71(c)										
3		20.406(a)(1)(ii)				50.38(e)(2)				50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)										
0		20.406(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(viii)(A)														
		20.406(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)														
		20.406(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(ix)														
LICENSEE CONTACT FOR THIS LER (12)																								
NAME Glenn B. Kirk, Compliance Section Engineer										TELEPHONE NUMBER AREA CODE 6 1 5 8 7 0 - 6 1 4 6														
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																								
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC														
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR								
YES (If yes, complete EXPECTED SUBMISSION DATE)												X NO												

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

At 2148 on 04/17/84, unit 1 experienced a reactor trip. Unit 1 was in mode 1 (2240 psig, 556 degrees F) at 30% reactor power just prior to the event. A turbine trip occurred due to failure of a generator stator cooling water pump. Subsequent low-low level in steam generator number three resulted in a reactor trip from approximately 18% reactor power. Unit 1 stabilized at 547 degrees F following the reactor trip.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 5/31/85

FACILITY NAME (1) Sequoyah, Unit 1	DOCKET NUMBER (2) 0500032784-026-0002 OF 02	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		

TEXT (If more space is required, use additional NRC Form 366A's) (17)

During a normal startup following a refueling outage on 04/17/84, unit 1 experienced a reactor trip. Just prior to the event which occurred at 2148 CST on 04/17/84, unit 1 was in mode 1 (2240 psig, 556 degrees F) at 30% reactor power.

The reactor trip occurred subsequent to a trip of the turbine generator from 30% power. Generator stator cooling water pump 'B' tripped due to a bearing failure resulting in a 'loss of generator cooling' turbine generator trip. Normally, when one stator cooling water pump fails, the opposite pump starts to supply cooling water, but the 'A' stator cooling pump had been removed from service during the outage and had not been returned to service. The reactor was maintained at power with the steam dump to condenser and the steam generator level control in manual. During the process of rolling the turbine, the already difficult task of manual steam generator level control was compounded by level fluctuations and isolations of the number two feedwater heaters and the turbine and feedwater pumps tripped on high-high level in the number two steam generator. Subsequent steam generator level shrink resulted in a reactor trip due to low-low level in the number three steam generator from approximately 18% power.

Unit 1 stabilized at 547 degrees F following the reactor trip. The generator stator cooling water pump 'A' had been returned to service during the attempt to restart the turbine and the stator cooling pump 'B' was replaced.

All other equipment and personnel performed as expected during the event. There was no effect on public health or safety.

For 1984, there have been three previous reactor trips on low-low steam generator level with steam generator level controls in manual, but this was the first occurrence in which the sequence of events was initiated by failure of a generator cooling water pump.

TENNESSEE VALLEY AUTHORITY

Sequoyah Nuclear Plant
Post Office Box 2000
Soddy Daisy, Tennessee 37379

May 15, 1984

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Gentlemen:

TENNESSEE VALLEY AUTHORITY - SEQUOYAH NUCLEAR PLANT UNIT 1 - DOCKET NO.
50-327 - FACILITY OPERATING LICENSE DPR-77 - REPORTABLE OCCURRENCE REPORT
SQRO-50-327/84026

The enclosed licensee event report provides details concerning a reactor trip from 18% power. This event is reported in accordance with 10 CFR 50.73, paragraph a.2.iv.

Very truly yours,

TENNESSEE VALLEY AUTHORITY



C. C. Mason
Power Plant Superintendent

Enclosure
cc (Enclosure):

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NRC Inspector, NUC PR, Sequoyah

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