

TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401  
400 Chestnut Street Tower II

May 14, 1984

Director of Nuclear Reactor Regulation  
Attention: Ms. E. Adensam, Chief  
Licensing Branch No. 4  
Division of Licensing  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Dear Ms. Adensam:

In the Matter of ) Docket No. 50-327  
Tennessee Valley Authority )

References: (1) L. M. Mills' letter to you dated March 23, 1984  
(2) L. M. Mills' letter to you dated April 13, 1984  
(3) L. M. Mills' letter to you dated April 13, 1984  
(4) My letter to you dated May 20, 1983

At the request of Carl Stahle of your staff, L. M. Mills' letter to you dated April 13, 1984 committed TVA to evaluate its listing of electrical equipment within the scope of 10 CFR 50.49 and provide within 30 days a reconfirmation that the listing is complete and accurate. As requested, TVA has now completed a review of its listing for equipment on unit 1, and we have identified eight additional components which are presently being tested to meet the requirements of NUREG-0588 and; therefore, have not been completed within the requirements of 10 CFR 50.49. The following enclosure provides additional information on these components. To the best of our knowledge with the exception of these eight items, TVA's listing of electrical equipment within the scope of 10 CFR 50.49 is now complete and accurate for Sequoyah unit 1.

If there are any questions, please get in touch with R. H. Shell at FTS 858-2688.

Very truly yours,

TENNESSEE VALLEY AUTHORITY

8405220249 840514  
PDR ADOCK 05000327  
P PDR

D S Kammer  
D. S. Kammer  
Nuclear Engineer

Sworn to and subscribed before me  
this 14<sup>th</sup> day of May 1984.

Paulette W. White  
Notary Public  
My Commission Expires 9-5-84

Enclosure  
cc: U.S. Nuclear Regulatory Commission (Enclosure)  
Region II  
Attn: Mr. James P. O'Reilly, Regional Administrator  
101 Marietta Street, NW, Suite 2900  
Atlanta, Georgia 30303

Aug 11

ENCLOSURE

NCR EEB8307, EEB8203

The eight components listed below are the steam generator level control valves installed on the auxiliary feedwater system. It was determined in late 1982 that the valve positioners (which convert a current signal to a pressure signal) on each of these air-operated valves lacked sufficient documentation to verify that they were qualified for the environmental conditions that could occur during certain accidents. Following this, TVA was unable to locate a qualified replacement positioner and finally concluded that this was the best available positioner.

Subsequently, these positioners were entered into a testing program to verify their qualification. This created further delays because of the time required to procure positioners from the manufacturer for testing. Initially the positioner was subjected to an accident simulation test to temperatures in excess of 325°F, which is greater than the device would encounter during an accident condition. The performance of the positioner was satisfactory during this test; however, to qualify the positioner to 323-1974 requirements completely, a long-term thermal aging, radiation, seismic, and subsequent LOCA test must be performed. This testing is still in progress and will not be completed until March 1985.

Justifications for continued operations (JCOs) were provided to you in our letter dated July 20, 1983, for these components. In addition, a similar I/P transducer which is installed on the auxiliary feedwater bypass level positioner (Masonellian model 8005A) control valves has been fully qualified by testing.

TVA believes the JCOs submitted to you are still applicable. Additionally, there is no indication at this time that the positioners in question will not pass the complete qualification testing program. Based on TVA's evaluation of all available information, we believe these positioners will function as intended. The following components are affected.

LCV-3-148, 156, 164, 171, 172, 173, 174, and 175.