

LICENSEE EVENT REPORT

CONTROL BLOCK:

(PLEASE PRINT ALL REQUIRED INFORMATION)

LICENSEE NAME				LICENSE NUMBER								LICENSE TYPE					EVENT TYPE							
01	I	L	D	R	S	3	0	0	-	0	0	0	0	-	0	0	4	1	1	1	1	0	3	
7	8	9				14	15									25	26					30	31	32

01		CON'T		CATEGORY		REPORT TYPE	L	REPORT SOURCE	L	DOCKET NUMBER	050-0249	EVENT DATE	102676	REPORT DATE	111876
7	8	57	58			59	60			61	68	69	74	75	81

EVENT	DESCRIPTION
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02 DURING REFUELING OUTAGE LOCAL LEAK-RATE TESTING, THE TIP PURGE
03 CHECK VALVE EXHIBITED A LEAKAGE RATE OF 188 SCFH (TECH SPEC LIMIT: 29.38)
04 SCFH). THE VALVE WAS REMOVED FROM THE SYSTEM AND CLEANED WITH
05 COMPRESSED AIR. FOLLOWING REINSTALLATION OF THE VALVE, TESTING YIELDED
06 A LEAKAGE RATE WELL BELOW THE PERMISSIBLE LIMIT. THIS IS THE FIRST
(SEE ATTACHED SHEET)

SYSTEM CODE		CAUSE CODE	COMPONENT CODE						PRIME COMPONENT SUPPLIER	COMPONENT MANUFACTURER				VIOLATION	
[0]	[7]	[I][F]	[E]	[V]	[A]	[L]	[V]	[E]	[X]	[N]	[X]	[9]	[9]	[9]	[Y]
7	8	9	10							43	44			47	48

CAUSE DESCRIPTION

08	THE VALVE LEAKAGE WAS APPARENTLY CAUSED BY DIRT ON THE BALL OR THE	80
09	SEATING AREA. AS STATED ABOVE, THE VALVE WAS CLEANED WITH COMPRESSED	80
10	AIR, WITH SATISFACTORY RESULTS. DURING REACTOR OPERATION, THE TIP	80
FACILITY METHOD OF		(SEE ATTACHED SHEET) 80

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

FACILITY STATUS: 11 H

% POWER: 000

OTHER STATUS: NA

METHOD OF DISCOVERY: B

DISCOVERY DESCRIPTION: NA

(SEE ATTACHED SHEET) 51

FORM OF ACTIVITY RELEASED CONTENT OF RELEASE AMOUNT OF ACTIVITY LOCATION OF RELEASE

12 Z NA NA

7 8 9 10 11 44 45 80

PERSONNEL EXPOSURES

NUMBER				TYPE	DESCRIPTION
1	3	0	0	0	NA

PERSONNEL INJURIES

NUMBER				DESCRIPTION
14	000			NA

OFFSITE CONSEQUENCES

15 | *NA* |

LOSS OR DAMAGE TO FACILITY

TYPE			DESCRIPTION
16	Z		NA
7	8	9	10
			8204140077 741118
			80

PUBLICITY
 17 NA
 7 8 9

ADDITIONAL FACTORS

18 NA

19
7 8 9

NAME: EDWARD L. SECKINGER

PHONE: EXT. 265

EVENT DESCRIPTION (Continued)

time this valve has been local leak-rate tested since unit operations began.
(50-249/1976-30)

CAUSE DESCRIPTION (Continued)

purge line is connected to the nitrogen makeup system, which is maintained at approximately 100 PSIG. Consequently, it is considered extremely improbable that any through-leakage would occur, even with drywell pressure at the design limit of 48 PSIG. The safety significance of this event is further minimized by the physical size of the purge line (1/4 - inch). To ensure the timely identification of a recurring problem, surveillance testing of the TIP purge check valves has been added to the refueling outage LLRT procedures for both Units 2 and 3. The valve is a 1/4 - inch ball check valve, model C400-10S, manufactured by Manetrol.



Commonwealth Edison
Dresden Nuclear Power Station
R.R. #1
Morris, Illinois 60450
Telephone 815/942-2920

1E FILE COPY



BBS Ltr. #76-815

November 22, 1976

Mr. James G. Keppler, Regional Director
Directorate of Regulatory Operations - Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137

Enclosed please find Reportable Occurrence report number 50-249/1976-30.
This report is being submitted to your office in accordance with the Dresden
Nuclear Power Station Technical Specifications, Section 6.6.B.

B. B. Stephenson
Station Superintendent
Dresden Nuclear Power Station

BBS:jo

Enclosure

cc: Director of Inspection & Enforcement
Director of Management Information & Program Control
File/NRC

12111