

NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) **SURRY POWER STATION - Unit 2** DOCKET NUMBER (2) **0 5 0 0 0 2 8 1 1** OF **0 3**

TITLE (4) **Reactor Trip (Vital Bus I Spike)**

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)									
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)								
0	4	1	5	8	4	8	4	0	0	9	0	5	0	0	0			
0	4	1	5	8	4	8	4	0	5	1	5	8	4	0	5	0	0	0

OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)									
POWER LEVEL (10)	0 1 0 2	20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)						
		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(e)						
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)						
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)							
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)							
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)							

LICENSEE CONTACT FOR THIS LER (12)		TELEPHONE NUMBER	
NAME	J. L. Wilson, Station Manager	AREA CODE	8 0 4 3 5 7 - 3 1 8 4

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)											
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	NO					

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

At 1607 on 4-15-84 following a maintenance outage, unit 2 was at 2% reactor power when a reactor trip occurred as a result of an intermediate range (NI-35) high flux trip. Plant parameters did not indicate a valid high flux trip.

An electrician was checking for continuity across the switch for TV-SS-201A when an arc occurred resulting in a spike on vital bus 1 which caused the spike on NI-35. The multimeter was selected to resistance instead of voltage.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
SURRY POWER STATION, Unit 2	0 5 0 0 0 2 8 1	8 4	0 0 9	0 0	0 2	OF	0 3

TEXT (If more space is required, use additional NRC Form 365A's) (17)

1. Description of the Event

At 1607 on 4-15-84 following a maintenance outage, unit 2 was at 2% reactor power when a reactor trip occurred as a result of an intermediate range (channel 35) (EIIS No. RI) high flux trip. Plant parameters did not indicate a valid high flux trip.

At the time of the trip, electricians were troubleshooting trip valve TV-SS-201A (EIIS No. SHV) which would not remotely close.

Immediately following the reactor trip, operators noted that all control and protection systems functioned properly.

2. Safety Consequences and Implications

Because of the low power condition of the plant, at the time of the trip, no significant transient ensued. A similar trip from full power would not have caused any safety limit to be exceeded since this event is analyzed in the UFSAR. For these reasons, an unreviewed safety question was not created and the health and safety of the public were unaffected.

3. Cause

An electrician was checking for continuity with a multimeter at the switch for TV-SS-201A when an arc occurred at the meter. This arc caused a voltage spike on vital bus 1 which resulted in the spike on intermediate range detector NI-35. Detector NI-35 is also fed from vital bus 1. Other nuclear instrumentation remained stable. Personnel error was the reason that the meter arced because it was selected for resistance instead of voltage for the initial reading.

4. Immediate Corrective Action

The Operators performed all appropriate emergency and function restoration procedures to ensure that plant conditions were stabilized. Also, the Shift Technical Advisor performed a review of the critical function status trees and completed the post trip review procedure to ensure specific plant parameters remained within safe bounds. The voltage spike did not result in loss of power to any safety related component, therefore, no further actions are required.

5. Additional Corrective Actions

None were required.

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SURRY POWER STATION, Unit 2	0 5 0 0 0 2 8 1 8 4 —	0	0	9	—	0	0
						0	3
						OF	0 3

TEXT (If more space is required, use additional NRC Form 365A's) (17)

6. Action Taken to Prevent Recurrence

The individual involved was reinstructed to utilize caution in dealing with safety equipment was reemphasized.

7. Generic Implications

This is not considered a generic event.

Vepco

VIRGINIA ELECTRIC AND POWER COMPANY

Surry Power Station
P. O. Box 315
Surry, Virginia 23883

MAY 18 1984

Serial No: 84-021

Docket No: 50-281

License No: DPR-37

U.S. Nuclear Regulatory Commission
Document Control Desk
016 Phillips Building
Washington, D. C. 20555

Gentlemen:

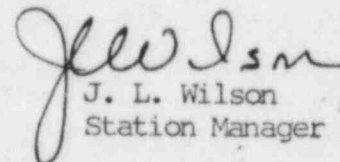
Pursuant to Surry Power Station Technical Specifications, the Virginia Electric and Power Company hereby submits the following Licensee Event Report for Surry Unit 2.

REPORT NUMBER

84-009-00

This report has been reviewed by the Station Nuclear Safety and Operating Committee and will be reviewed by Safety Evaluation and Control.

Very truly yours,


J. L. Wilson
Station Manager

Enclosure

cc: Mr. James P. O'Reilly
Regional Administrator
Suite 2900
101 Marietta Street, NW
Atlanta, Georgia 30303

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