

BEFORE THE  
UNITED STATES NUCLEAR REGULATORY COMMISSION

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DOCKET NOS. 50-416 AND 50-417

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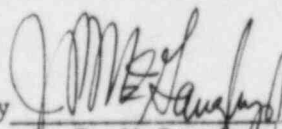
IN THE MATTER OF  
MISSISSIPPI POWER & LIGHT COMPANY  
AND  
MIDDLE SOUTH ENERGY, INC.  
AND  
SOUTH MISSISSIPPI ELECTRIC POWER ASSOCIATION

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AMENDMENT NO. 58 TO  
FINAL SAFETY ANALYSIS REPORT

Mississippi Power & Light Company for itself and on behalf of Middle South Energy, Inc. and South Mississippi Electric Power Association herewith files this Amendment No. 58 to their Final Safety Analysis Report in the form of revisions to certain sections of the Final Safety Analysis Report.

Respectfully submitted  
Mississippi Power & Light Company

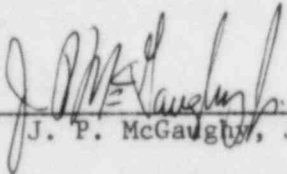
By   
J. P. McGaughy, Jr.  
Vice President - Nuclear

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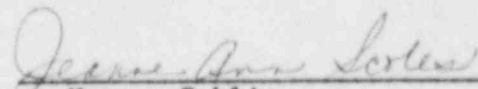
STATE OF MISSISSIPPI  
COUNTY OF HINDS

J. P. McGaughy, Jr., being duly sworn, states that he is Vice President - Nuclear, of Mississippi Power & Light Company; that he is authorized on the part of said Company to sign and file with the Nuclear Regulatory Commission this Amendment No. 58 to the Final Safety Analysis Report on behalf of said Company, Middle South Energy, Inc., and South Mississippi Electric Power Association; that he signed the foregoing amendment as Vice President - Nuclear of Mississippi Power & Light Company; and that the statements made and the matters set forth therein are true and correct to the best of his knowledge, information, and belief.

  
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J. P. McGaughy, Jr.

SUBSCRIBED AND SWORN TO before me, a Notary Public, in and for the County and State above named, this 14th day of May, 1984.

(SEAL)

  
\_\_\_\_\_  
Notary Public

My Commission Expires Sep. 21, 1987

Table 1

Description of FSAR Amendment 58 Changes  
Related to Technical Specification Problem Sheets (TSPS)\*

<u>TSPS</u>	<u>Description of FSAR Change</u>
001	Revise small break accident analysis in subsection 6.3.3.7.8, Table 6.3-2, and in the response to Question 212.24 to include the assumption that one ADS valve is out of service. Add new Figures 6.3-77 through 6.3-85 and 212.24-7 through 212.24-12 accordingly.
028	Provide more specific valve designations in the response to Question 110.48.
033	Revise subsection 7.3.1.1.4.4 to reflect the deletion of the 90 second time delay between actuation of containment sprays A and B. Clarify accident analysis in subsection 6.2.1.1.4.2 accordingly.
102	Revise Figure 9A-22 to reflect updated fire zones in diesel generator buildings.
103	Revise Table 7.3-10 to reflect that there are eight main steam line high flow sensor channels per trip system.
131	Revise Figure 9.5-4 to clarify that the hose station locations given in the figure are approximate and to correct a typographical error.
147	Revise subsection 8.3.1.1.3 to reflect the Technical Specifications; i.e., for undervoltage protection of the ESF busses during a loss of preferred power event, the minimum and maximum values of voltage setpoints and time delays are applied to the allowable values and not to the trip setpoints.
151 (& 800)	Revise APRM system trips and setpoint ranges in Tables 7.2-1 and 7.6-6 to be consistent with Technical Specification values.
152	Revise control rod surveillance description in subsection 4.6.3.1.1.5 to be consistent with the Technical Specifications and actual plant procedures.
172	Revise subsection 3.8.3.1.1 to update preliminary drywell design basis leakage parameters.  Revise subsection 6.2.1.1.5.4 concerning drywell leak rate test schedules.

\*Ref: AECM-84/0217, dated April 9, 1984, "GGNS Technical Specification Review Program"

TSPSDescription of FSAR Change

- Revise subsections 6.2.6.3, 6.2.6.4, and 6.2.6.5 to change references to "Chapter 16" to "Technical Specifications" and to provide clarifications of drywell leakage and preoperational tests.
- 202 Revise section 7.5 to clarify the suppression pool temperature monitoring system's normal and post-accident power supply. Delete obviated system descriptions from section 7.6.
- 234 Revise subsection 6.2.7.5 to clarify the description of the suppression pool level instrumentation and revise Figures 1.2-8 and 3.8-60 to show the actual suppression pool normal low water level.
- 251 Update the SRM downscale trip setpoint in Table 7.6-3 to reflect the change made to the Technical Specifications in O.L. Amendment 12.
- 252 Revise subsection 6.2.1.1.4.2 and tables 1.3-4, 6.2-1, 6.5-6 to indicate the volume of the containment to be 1,400,000 cubic feet and the drywell volume to be 270,000 cubic feet.
- 259 Revise table 5.2-11 to show that the RCPB was designed for 80 thermal cycles.
- 260 Consistent with AECM-82/353, dated August 19, 1982, revise subsection 6.2.1.1.4.2 to add reference to containment negative pressure analyses conducted as part of the Mark III Containment Issues Program (Humphrey Concerns) to support the 90°F technical specification containment air temperature limit. Revise Table 3.11-1 accordingly. Revise Table 6.2-4 to reflect the 95°F temperature used for all other analyses.
- 291 Revise Table 6.2-44, sheet 24 of 27, to clarify that the 5.5 second MSIV closure time is composed of a 5.0 second actual valve closure time plus 0.5 seconds for instrument response time.
- 300 Correct a typographical error in the MAPLHGR vs. Fuel Exposure list given in Table 6.3-6.
- 335 Revise Appendix 3A to indicate compliance with Regulatory Guide 1.137, January 1978, consistent with the response to Question 040.44 previously provided by FSAR Amendment 51.
- 339 Provide cross-reference in section 18.1.3 to the FSAR section where definition of non-licensed operator is provided.
- 340 Revise the operator overtime restrictions given in subsection 13.1.2.1 to be consistent with section 18.1 and the Technical Specifications.

<u>TSPS</u>	<u>Description of FSAR Change</u>
800	See TSPS 151.
801	Revise the allowable containment normal differential pressure in Table 3.11-1 to be consistent with the Technical Specifications.
802	Revise the description of the recirculation pump trip system contained in subsection 7.6.1.8.1 to be consistent with an FSAR Amendment 55 revision of the same system's description contained in FSAR section 15.2.
805	Revise Figure 9.3-26 to reflect updated design information for the volume of sodium pentaborate required in the SLCS storage tank.
806	Revise subsection 15.4.7 to reflect the latest General Electric generic analysis for the misplaced bundle accident.
807	Revise the descriptions of radiation monitor channel check and channel functional test contained in subsection 11.5.2.3.1 to be consistent with the Technical Specifications.
809	Revise subsection 7.1.2.6.22 to more clearly discuss the methods of bypassing the thermal overload protection devices of motor operated valves.
810	Revise the description of periodic testing of low voltage circuit breakers contained in the response to Question 040.5 to reference the Technical Specifications.
812	Revise subsection 7.6.1.4.3.3.2.1 to clarify that the logic that provides the isolation signal to RCIC for main steam tunnel high temperature includes a time delay.
816	Revise Table 7.3-10 to update instrument ranges and allowable setpoint ranges.
817	Revise subsection 6.2.3.2 to clarify SGTS drawdown capability.
818	Revise subsection 6.2.3.2 to reflect the use of blind flanges and rupture discs to maintain secondary containment isolation.
819	Revise Table 3.7-17 concerning deletion of triaxial response recorders to be consistent with the text of subsection 3.7.4.2.5 and the SER. Add miscellaneous editorial changes and clarifications to same.
820	Consistent with the response to Question 211.120, delete RCIC trip settings from Table 7.4-1.

<u>TSPS</u>	<u>Description of FSAR Change</u>
821	Delete reference to verificaiton of CRD accumulator water level contained in subsection 4.6.3.1.1.5.
822	Revise subsection 6.2.3.1.1 to be consistent with subsection 6.5.3.2 regarding 120 second time period to achieve specified negative pressure in the secondary containment.
823	Consistent with O.L. Amendment 9, revise Table 7.6-12 to identify valve E12-F203 as an auxiliary building isolation valve.
824	Revise Table 6.2-44 to reflect the Technical Specifications, as-built plant, and miscellaneous updates and editorial clarifications.
828	Make editorial changes to subsections 7.4.1.1.3.2 and 7.4.1.1.3.5 concerning low water setpoints.
830	Revise subsection 5.3.3.6 to update the reactor coolant temperature differential between the reactor dome and bottom head drain.
831	Revise Table 7.2-5, the response to Question 211.134, and subsection 7.2.1.1.4 concerning RPS response time to be consistent with AECM-82/142 dated April 30, 1982.
832	Clarify description of manual initiation of SGTs contained in subsection 7.3.1.1.8.2.
833	Revise Tables 8.3-1, 8.3-2, and 8.3-3 to reflect actual maximum loads based on preoperational test data.
**	Revise Table 1.3-1 to indicate that the weight of one fuel assembly (including channel) is 699 lb.
**	Revise subsection 8.3.1.1.4.2.10 to clarify HPCS diesel generator trips.

\*\*Not initiated by a TSPS.