



Commonwealth Edison
Quad-Cities Nuclear Power Station
Post Office Box 216
Cordova, Illinois 61242
Telephone 309/654-2241

IE FILE COPY

NJK-76-386

October 22, 1976

J. Keppler, Regional Director
Office of Inspection and Enforcement
Region III
U.S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137

Reference: Quad-Cities Nuclear Power Station
Docket No. 50-265, DPR-30, Unit 2
Appendix A, Sections 3.6.1.4., 6.6.B.2.b

Enclosed please find Reportable Occurrence Report No. RO 50-265/76-14 for Quad-Cities Nuclear Power Station.

This report is submitted to you in accordance with the requirements of Technical Specification 6.6.B.2.

Very truly yours,

COMMONWEALTH EDISON COMPANY
QUAD-CITIES NUCLEAR POWER STATION

N. J. Kalivianakis
Station Superintendent

NJK/TPJ/vmb

cc: G. A. Abrell

8306130149 761022
PDR ADOCK 05000265
S PDR

OCT 25 1976

11440

LICENSEE EVENT REPORT

CONTROL BLOCK: 1 6

[PLEASE PRINT ALL REQUIRED INFORMATION]

LICENSEE NAME										LICENSE NUMBER										LICENSE TYPE					EVENT TYPE		
01	1	1	L	Q	A	D	2			0	0	-	0	0	0	0	-	0	0	4	1	1	1	1	0	3	
7	8	9					14			15										25	26				30	31	32

CATEGORY		REPORT TYPE	REPORT SOURCE	DOCKET NUMBER					EVENT DATE					REPORT DATE										
01	CON'T		L	L	0	5	0	-	0	2	6	5	0	9	2	2	7	6	1	0	2	2	7	6
7	8		57	58	59	60	61			68	69			74					75					80

EVENT DESCRIPTION

02	During routine inspection surveillance of Unit 2 Hydraulic Snubbers on September 22,																								80
03	1976, three Grinnell snubbers and one Bergen-Patterson snubber were found to have																								80
04	empty oil reservoirs. Work request 3529-76 was issued to repair and refill these																								80
05	defective snubbers. All other snubbers that were inspected on Unit 2 had satisfactory																								80
06	fluid levels. The following snubbers were found empty: See Attached																								80

SYSTEM CODE			CAUSE CODE		COMPONENT CODE					PRIME COMPONENT SUPPLIER		COMPONENT MANUFACTURER					VIOLATION	
07	X	X	E		H	A	N	G	E	R	A	G	2	5	7	N		
7	8	9	10	11	12					17	43	44	B	2	1	0	47	48

CAUSE DESCRIPTION

08	(Proximate Cause-Equipment Failure) Upon inspection, all four snubbers were found to																								80
09	have oil leakage around the shafts through the "O" rings. The rings were observed to																								80
10	be dried out and cracked in several places.																								80

FACILITY STATUS			% POWER			OTHER STATUS			METHOD OF DISCOVERY			DISCOVERY DESCRIPTION												
11	H	0	0	0		NA	B	Routine Inspection Surveillance																
7	8	9	10	11	12	13	44	45	46															80

FORM OF ACTIVITY RELEASED			CONTENT OF RELEASE			AMOUNT OF ACTIVITY			LOCATION OF RELEASE															
12	Z	Z				NA				NA														
7	8	9	10	11	12	13	44	45																80

PERSONNEL EXPOSURES

NUMBER			TYPE		DESCRIPTION																			
13	0	0	0	Z	NA																			
7	8	9	11	12	13																			80

PERSONNEL INJURIES

NUMBER			DESCRIPTION																					
14	0	0	0	NA																				
7	8	9	11	12																				80

OFFSITE CONSEQUENCES

15	NA																								80
----	----	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	----

LOSS OR DAMAGE TO FACILITY

TYPE			DESCRIPTION																						
16	Z	NA																					80		
7	8	9	10																						80

PUBLICITY

17	NA																								80
----	----	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	----

ADDITIONAL FACTORS

18																									80
----	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	----

19																									80
----	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	----

NAME: Thomas Joyce

PHONE: 309-654-2241 Ext. 247

Grinnell Serial No. 4754: Located on line 2-1307-3", Reactor Core Isolation Cooling (RCIC) System steam supply.

Grinnell Serial No. 4750: Located on line 2-1202-6", Reactor Water Cleanup System suction.

Grinnell Serial No. 4753: Located on line 2-1202-6", Reactor Water Cleanup System suction.

Bergen-Patterson Mark No. 180: Located on Residual Heat Removal (RHR) System "B" return loop to the reactor vessel.

Hydraulic snubbers are designed to prevent unrestrained pipe motion under dynamic loads as might occur during an earthquake or severe transient. Since snubbers are only required to operate during low-probability events, a time period of 72 hours is allowed for repair or replacement while the unit is operating at power. Since these snubbers did not result in any system piping failure, and did not cause any components to be rendered inoperable, the safety implications of this occurrence are minimal. Since there were no releases to the environs, the health and safety of the public were not effected due to this occurrence.

CORRECTIVE ACTION TO PREVENT RECURRENCE

Each of the failed snubbers was removed and subsequently rebuilt with new "O" rings. The fluid reservoirs of these snubbers were refilled, and no leakage was observed. The snubbers were then re-installed in their designated locations. This corrective action should be adequate to prevent recurrence of future leakage problems with respect to these snubbers.

FAILURE DATA

There have been previous problems with leaking hydraulic snubbers. None of these failures have resulted in any hazard to the public nor jeopardized the safe operation of the facility. The purpose of the periodic inspection is to discover the problems and perform corrective action on a timely basis, so continued safe plant operation may take place.