

UPDATE REPORT
PREVIOUS REPORT DATE: 9/22/75

[PLEASE PRINT ALL REQUIRED INFORMATION]

01		CONT		CATEGORY		REPORT TYPE		REPORT SOURCE		DOCKET NUMBER				EVENT DATE				REPORT DATE							
7	8	57	58	59	60	61	62	63	64	65	66	67	68	69	70	71	72	73	74	75	76	77	78		
				L	L	0	5	0	-	0	2	4	9	0	9	2	2	7	6	1	2	2	9	7	6

02 DURING REFUELING OUTAGE LOCAL LEAK-RATE TESTING, THE VOLUME BOUNDED
7 8 9 61

03 BY MAIN STEAM LINE DRAIN VALVES 3-220-1 AND -2 EXHIBITED A LEAKAGE
7 8 9 60

04 RATE OF 33.5 SCFH (TECH SPEC LIMIT: 29.381 SCFH). THIS LEAKAGE
7 8 9 60

05 APPEARED TO BE THE RESULT OF A VALVE BODY-TO-BONNET LEAK ON
7 8 9 60

06 3-220-1, THE INBOARD VALVE. VALVE 3-220-1 WAS DISASSEMBLED, AND
7 8 9 60

PRIME (SEE ATTACHED SHEET)

SYSTEM CODE				CAUSE CODE	COMPONENT CODE						PRINT COMPONENT SUPPLIER	COMPONENT MANUFACTURER				VIOLATION
0	7	C	C	E	V	A	L	V	E	X	A	C	6	6	5	Y
7	8	9	10	11	12					17	43	44			47	48

08	THE LEAKAGE WAS CAUSED BY A DETERIORATED BODY-TO-BONNET PRESSURE	80
09	SEAL RING, AND BY DIRT IN THE SEATING AREA ON VALVE 3-220-1. RE-	80
10	PLACING THE SEAL RING AND LAPPING THE VALVE DISC AND SEAT	80
EACH OF		METHOD OF
		(SEE ATTACHED SHEET)

FACILITY STATUS		% POWER	OTHER STATUS	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION
11	H	000	NA	B	NA
7 8	9	10 12	13	44 45	46 80

FORM OF ACTIVITY RELEASED: [Z] CONTENT OF RELEASE: [Z] AMOUNT OF ACTIVITY: NA LOCATION OF RELEASE: NA

NUMBER				TYPE	DESCRIPTION
1	3	0	0	0	Z NA

NUMBER				DESCRIPTION
1	4			NA

15 | NA | 60

TYPE		DESCRIPTION
16	2	NA

7 8 9 NA 000-100011-7-1001 FIG

19 NA S PDR

8306100344 761021
PDR ADDCK 05000249
S PDR

19 7 8 9 80

EVENT DESCRIPTION (Continued)

the pressure seal ring between the body and bonnet was replaced; however, retesting of the volume bounded by the two valves again yielded unsatisfactory results. Valve 3-220-1 was disassembled once more, and the disc and seat were lapped. A subsequent LLRT yielded a leakage rate well below the Tech Spec limit.

Because this valve is located inside the drywell, any leakage would have been contained within the primary containment. This was the first reported leak-rate test failure for these valves. (50-249/1976-20)

CAUSE DESCRIPTION (Continued)

resulted in a greatly reduced leakage rate. Local leak-rate testing of these valves each refueling outage is adequate to identify deficiencies in a timely manner. Valve 3-220-1 is a 2-inch globe valve model 7852-U, manufactured by the Crane Company.



Commonwealth Edison
Dresden Nuclear Power Station
R.R. #1
Morris, Illinois 60450
Telephone 815/942-2920

BBS Ltr. #76-900

December 29, 1976

Mr. James G. Keppler, Regional Director
Directorate of Regulatory Operations - Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137

Enclosed please find an update report to Reportable Occurrence report number 50-249/1976-20. This report is being submitted to your office in accordance with the Dresden Nuclear Power Station Technical Specifications, Section 6.6.B.

B. B. Stephenson
Station Superintendent
Dresden Nuclear Power Station

BBS:jo

Enclosure

cc: Director of Inspection & Enforcement
Director of Management Information & Program Control
File/NRC

ADMINISTRATIVE PROCEDURES
(APPENDIX B)

Q.C.P. 15-52.2
Revision 0
September 1974

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SUPPLEMENT TO DVR

50-249

DVR NO.	STA	UNIT	YEAR	NO.
D - 12	- 3	- 76	- 44	

PART 1	TITLE OF EVENT	OCCURRED
	Excessive Steam Drain Valve Leakage	9/22/76 1300
		DATE TIME
REASON FOR SUPPLEMENTAL REPORT		
Disassembly and repair of valve 3-220-1 was not practicable at the time of initial report -		
PART 2		
ACCEPTANCE BY STATION REVIEW	C. Dargent	R. L. Lagan
DATE	12/29/76	12/29/76
SUPPLEMENTAL REPORT APPROVED AND AUTHORIZED FOR DISTRIBUTION	[Signature]	12-29-76
	STATION SUPERINTENDENT	DATE

1976-20

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III



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APPROVED

SEP 24 1974

D.C.S.R.