

CONTROL BLOCK:

LICENSEE
NAME

LICENSE NUMBER

LICENSE
TYPE

EVENT
TYPE

NAME					EXHIBIT NUMBER					TYPE					TYPE					
01	I	L	D	R	S	3	00	-	00	00	00	-	00	4	1	1	1	1	0	3
7	8	9			14	15							25	26					31	32

01 CONT		CATEGORY		REPORT TYPE	REPORT SOURCE	DOCKET NUMBER				EVENT DATE				REPORT DATE											
7	8	57	58	59	60	0	5	0	-	0	2	4	9	1	0	0	7	7	6	1	1	0	4	7	6

02 DURING REFUELING OUTAGE LOCAL LEAK-RATE TESTING, LPCI PRIMARY
7 8 9 80

03 ISOLATION CHECK VALVE 3-1501-25B EXHIBITED A LEAKAGE RATE IN
7 8 9 80

04 EXCESS OF THE TECH SPEC LIMIT OF 29.381 SCFH. THE OTHER VALVE
7 8 9 80

05 USED TO FORM THE 48-PSIG PRESSURE DECAY TEST BOUNDARY, MANUAL
7 8 9 80

06 VALVE 3-1501-26B, HAD A SIGNIFICANT WATER HEAD PRESENT; CONSEQUENTLY,
7 8 9 80

PRIME (SEE ATTACHED SHEET)

SYSTEM CODE		CAUSE CODE	COMPONENT CODE					COMPONENT SUPPLIER	COMPONENT MANUFACTURER			(SEE VIOLATION)			
0	7	S F	E	V	A	L	V	E	X	N	A	5	8	5	Y
7	8	9	10	11	12				17	43	44			47	48

08	THE VALVE FAILURE WAS CAUSED BY EXCESSIVE DISC-TO-SEAT LEAKAGE.				80
09	THIS LEAKAGE WAS ATTRIBUTED TO THE PRESENCE OF DIRT AND SMALL				80
10	SCRATCHES ON THE VALVE SEAT, APPARENTLY THE RESULT OF NORMAL WEAR.				80
	FACILITY STATUS	% POWER	OTHER STATUS	METHOD OF DISCOVERY	(SEE ATTACHED SHEET) DISCOVERY DESCRIPTION

FACILITY STATUS		% POWER		OTHER STATUS		METHOD OF DISCOVERY		(SEE ATTACHED SHEET) DISCOVERY DESCRIPTION	
11	H	1000		NA		B		NA	
7	8	9	10	11	12	13	44	45	46
FORM OF ACTIVITY RELEASED		CONTENT OF RELEASE		AMOUNT OF ACTIVITY				LOCATION OF RELEASE	
12	Z	10	Z		NA			NA	
7	8	9	10	11	44	45			80

NUMBER				TYPE	DESCRIPTION
1	3	0	0	0	Z NA

NUMBER				DESCRIPTION	
1	4	0	0	0	NA

1	5	NA										
7	8	9	80									

TYPE		DESCRIPTION
16	Z	NA

17	NA	PDR	ADDER	000000	CE77	PDR
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7 8 9 _____ 80

10

NAME: RANDOLPH C. WEIDNER

PHONE: EXT. 265

EVENT DESCRIPTION (Continued)

the as-found leakage of 61.0 SCFH was attributed entirely to leakage through check valve 1501-25B. A second leak test using valves 1501-26B, 1501-22B, and 1001-5B as boundaries yielded a leakage rate of 4.2 SCFH. Since the second test volume included the volume of the previous test, total through-leakage for the penetration was well under the Tech Spec limit. A similar valve on Unit-2 failed local leak-rate testing in early 1975. (50-249/1976-23)

CAUSE DESCRIPTION (Continued)

The valve was disassembled, the seat was lapped, and the packing and Flexitallic gasket were replaced. Leak-rate testing of the reassembled valve was satisfactory.

In 1975, a test/drain line was installed on this penetration which permitted leak-rate testing of check valve 3-1501-25B for the first time since unit operation began. Since this valve has performed satisfactorily over a period of several years, testing of this valve on an approximately annual basis should permit timely identification of a recurring problem. Valve 3-1501-25B is a 16-inch tilting disc check valve, manufactured by Atwood and Morrill Company, Inc.



Commonwealth Edison
Dresden Nuclear Power Station
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Morris, Illinois 60450
Telephone 815/942-2920

IE FILE COPY



BBS Ltr. #76-775

November 4, 1976

Mr. James G. Keppler, Regional Director
Directorate of Regulatory Operations - Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137

Enclosed please find Reportable Occurrence report number 50-249/1976-23.
This report is being submitted to your office in accordance with the Dresden
Nuclear Power Station Technical Specifications, Section 6.6.B.

Arthur M. Roberts
for B. B. Stephenson
Station Superintendent
Dresden Nuclear Power Station

BBS:jo

Enclosure

cc: Director of Inspection & Enforcement
Director of Management Information & Program Control
File/NRC

11462