

# LICENSEE EVENT REPORT

CONTROL BLOCK: 1 2 3 4 5 6

(PLEASE PRINT ALL REQUIRED INFORMATION)

LICENSEE NAME 01 I L D R S 3														LICENSE NUMBER 00-000000-00											LICENSE TYPE 41111					EVENT TYPE 03								
CATEGORY 01 CONT														REPORT TYPE L		REPORT SOURCE L		DOCKET NUMBER 050-0249											EVENT DATE 1100776					REPORT DATE 1110476				

## EVENT DESCRIPTION

02 DURING REFUELING OUTAGE LOCAL LEAK-RATE TESTING, FEEDWATER CHECK																																																																															
03 VALVES 3-220-58B AND -62B EXHIBITED LEAKAGE RATES OF 437 AND																																																																															
04 612 SCFH RESPECTIVELY (TECH SPEC LIMIT: 29.381 SCFH). ADDITIONALLY, THE																																																																															
05 LEAKAGE RATES EXHIBITED BY THESE TWO VALVES CAUSED THE TOTAL LEAKAGE																																																																															
06 RATE FOR ALL TESTABLE PENETRATIONS AND ISOLATION VALVES TO EXCEED																																																																															
(SEE ATTACHED SHEET)																																																																															

SYSTEM CODE 07 C H										CAUSE CODE E		COMPONENT CODE V A L V E X						PRIME COMPONENT SUPPLIER A		COMPONENT MANUFACTURER C 6 6 5						VIOLATION Y	
-----------------------	--	--	--	--	--	--	--	--	--	-----------------	--	-------------------------------	--	--	--	--	--	-------------------------------	--	-----------------------------------	--	--	--	--	--	----------------	--

## CAUSE DESCRIPTION

08 THE LEAKAGE THROUGH EACH CHECK VALVE WAS APPARENTLY CAUSED																																																																															
09 BY DIRT IN THE SEATING AREA, WHICH RESULTED IN EXCESSIVE DISC-																																																																															
10 TO-SEAT CLEARANCES. PRESSURIZING THE VALVES WITH WATER REDUCED																																																																															
(SEE ATTACHED SHEET)																																																																															

FACILITY STATUS 11 H										% POWER 000				OTHER STATUS NA										METHOD OF DISCOVERY B										DISCOVERY DESCRIPTION NA									
FORM OF ACTIVITY RELEASED 12 Z										CONTENT OF RELEASE Z										AMOUNT OF ACTIVITY NA										LOCATION OF RELEASE NA													

## PERSONNEL EXPOSURES

NUMBER 13 000										TYPE Z		DESCRIPTION NA									
------------------	--	--	--	--	--	--	--	--	--	-----------	--	-------------------	--	--	--	--	--	--	--	--	--

## PERSONNEL INJURIES

NUMBER 14 000										DESCRIPTION NA									
------------------	--	--	--	--	--	--	--	--	--	-------------------	--	--	--	--	--	--	--	--	--

## OFFSITE CONSEQUENCES

15 NA																																																																															
-------	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--

## LOSS OR DAMAGE TO FACILITY

TYPE 16 Z										DESCRIPTION NA									
--------------	--	--	--	--	--	--	--	--	--	-------------------	--	--	--	--	--	--	--	--	--

## PUBLICITY

17 NA																																																																															
-------	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--

## ADDITIONAL FACTORS

18 NA																																																																															
-------	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--

19																																																																															
----	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--

8306100320 770201  
PDR ADOCK 05000249  
S PDR

NAME: EDWARD L. SECKINGER

PHONE: EXT. 265

#### EVENT DESCRIPTION (Continued)

the Tech Spec limit. A few days later, these check valves were pressurized with water to 500 PSIG in an attempt to more closely approximate accident conditions. Subsequent retesting of valves 3-220-58B and -62B yielded leakage rates of 15.9 and 29.7 SCFH respectively. Since the leakage through valve 3-220-58B was reduced to a value below Tech Spec limits, no further action was necessary. Valve 220-62B was disassembled and inspected.

Redundant valves were available and operable during unit operation. This is a repetitive occurrence. (50-249/1976-25)

#### CAUSE DESCRIPTION (Continued)

these clearances and resulted in much lower leakage rates.

Disassembly of valve -62B revealed no defective components; the only abnormality was dirt in the valve seat. The valve was cleaned, and the disc assembly and seal ring were replaced as a precautionary measure. Following reassembly, valve 220-62B was retested, and a leak rate of 1.03 SCFH was obtained.

Because of the high incidence of initial LLRT failures on the feedwater check valves, the station has reevaluated the present method of testing these valves. A new method has been developed which more closely represents the normal safety-related operation of these valves during postulated accidents. After this new testing procedure has been adequately researched and evaluated, a letter will be submitted delineating the station's proposal.

Both valves are 18-inch tilting disc check valves manufactured by the Crane Company.



Commonwealth Edison  
Dresden Nuclear Power Station  
R.R. #1  
Morris, Illinois 60450  
Telephone 815/942-2920

IE FILE COPY

BBS Ltr. #76-776

November 4, 1976



Mr. James G. Keppler, Regional Director  
Directorate of Regulatory Operations - Region III  
U. S. Nuclear Regulatory Commission  
799 Roosevelt Road  
Glen Ellyn, Illinois 60137

Enclosed please find Reportable Occurrence report number 50-249/1976-25.  
This report is being submitted to your office in accordance with the Dresden  
Nuclear Power Station Technical Specifications, Section 6.6.B.

*Arthur M. Roberts*

for B. B. Stephenson  
Station Superintendent  
Dresden Nuclear Power Station

BBS:jo

Enclosure

cc: Director of Inspection & Enforcement  
Director of Management Information & Program Control  
File/NRC

11454