

LICENSEE EVENT REPORT

CONTROL BLOCK:

PLEASE PRINT ALL REQUIRED INFORMATION.

LICENSEE NAME						LICENSE NUMBER						LICENSE TYPE					EVENT TYPE			
01	I	L	D	R	S	3	00	-	00	00	00	-	00	4	1	1	1	1	0	3
7	8	9			14	15						25	26					31	32	

CATEGORY		REPORT TYPE	REPORT SOURCE	DOCKET NUMBER				EVENT DATE				REPORT DATE											
01	CONT	L	L	0	5	0	-	0	2	4	9	1	0	2	8	7	6	1	1	2	6	7	6
7	8	57	58	59	60	61	62	63	64	65	66	67	68	69	70	71	72	73	74	75	76	77	

EVENT DESCRIPTION

02 DURING ROUTINE REFUELING OUTAGE SURVEILLANCE TESTING, CRD RETURN
7 8 9

03 LINE PRIMARY ISOLATION CHECK VALVE 3-0301-98 EXCEEDED THE
7 8 9

04 TECH SPEC LEAK-RATE LIMIT OF 29.38 SCFH. THE OTHER VALVE USED
7 8 9

05 TO FORM THE 48-PSIG PRESSURE DECAY TEST BOUNDARY WAS MANUAL
7 8 9

06 VALVE 3-0301-99. SINCE THE VESSEL WATER HEAD WAS PRESENT ON
7 8 9

SYSTEM CODE				CAUSE CODE	COMPONENT CODE				PRIME COMPONENT SUPPLIER	COMPONENT MANUFACTURER				VIOLATION		
0	7	R	B	E	V	A	L	V	E	X	N	V	I	3	5	Y
7	8	9	10	11	12					17	43	44			47	48

CAUSE DESCRIPTION

08 EXCESSIVE DISC-TO-SEAT LEAKAGE RESULTED FROM A COMBINATION OF
09 TWO FACTORS. FIRST, ALTHOUGH THE VALVE'S SEATING SURFACE WAS IN
09 EXCELLENT CONDITION, NORMAL WEAR HAD APPARENTLY REDUCED THE

FACILITY STATUS		% POWER		OTHER STATUS		METHOD OF DISCOVERY		DISCOVERY DESCRIPTION	
11	H	000	NA	B	NA				
7 8	9	10 12	13	44	45	46			

FORM OF ACTIVITY RELEASED		CONTENT OF RELEASE		AMOUNT OF ACTIVITY		LOCATION OF RELEASE	
12	[Z]	[Z]	NA	NA			
7	8	9	10	11	44	45	

PERSONNEL EXPOSURES

NUMBER			TYPE	DESCRIPTION
1	3	0 0 0	Z	NA
7	8	9	11	12
				13

PERSONNEL INJURIES

NUMBER			DESCRIPTION
1	4		NA

OFFSITE CONSEQUENCES

15 LNA

LOSS OR DAMAGE TO FACILITY

TYPE		DESCRIPTION
10	Z	NA

PUBLICITY

1	7	NA
7	8	9

8306100299 761126
PDR ADCK 05000249
S PDR

ADDITIONAL FACTORS

19	NA
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1	2	
7	8	9

NAME: RANDOLPH C WEIDNER

PHONE: EXT. 265

EVENT DESCRIPTION (Continued)

the other side of the manual valve, it was assumed to be leak-tight, and the resulting as-found leakage of 35.0 SCFH was attributed entirely to leakage through check valve 3-0301-98. A second leak test using valves 0301-99 and 0301-95 (a check valve in series with valve 0301-98) as boundaries yielded a leakage rate of 10.2 SCFH. Since the second test volume included the volume of the previous test, total through-leakage for the penetration was well under the Tech Spec limit. A similar valve on Unit-2 failed in May, 1975. (50-249/1976-29)

CAUSE DESCRIPTION (Continued)

total seating area sufficiently to allow some leakage to occur. Secondly, the valve piston had light surface rust on all non-seating surfaces, which evidently caused occasional binding of the piston in its bore.

The valve was disassembled and the seat was lapped. The valve piston was cleaned to remove surface rust, and then was lubricated to ensure that the piston remained rust-free until the CRD system was returned to service. Leak-rate testing of the reassembled valve was satisfactory.

During the 1976 refueling outage, a test/drain line was installed on this penetration which permitted leak-rate testing of check valve 3-0301-98 for the first time since unit operation began. Since valve 0301-98 has performed satisfactorily over a period of several years, and since its identical back-up check valve (0301-95) continues to perform satisfactorily, testing of this valve on an approximately annual basis should permit timely identification of any recurring problem.

Valve 3-0301-98 is a 4-inch piston-type lift check valve, manufactured by Henry Vogt Machine Co.



Commonwealth Edison
Dresden Nuclear Power Station
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Telephone 815/942-2920

TE FILE COPY

BBS Ltr. #76-823

November 26, 1976



Mr. James G. Keppler, Regional Director
Directorate of Regulatory Operations - Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137

Enclosed please find Reportable Occurrence report number 50-249/1976-29.
This report is being submitted to your office in accordance with the Dresden
Nuclear Power Station Technical Specifications, Section 6.6.B.

B. B. Stephenson
Station Superintendent
Dresden Nuclear Power Station

BBS:jo

Enclosure

cc: Director of Inspection & Enforcement
Director of Management Information & Program Control
File/NRC

12148