

LICENSEE EVENT REPORT

CONTROL BLOCK: 1 6

PLEASE PRINT ALL REQUIRED INFORMATION

LICENSEE NAME	LICENSE NUMBER	LICENSE TYPE	EVENT TYPE
01 I L D R S 3	0 0 - 0 0 0 0 0 - 0 0	4 1 1 1 1 1	0 3
7 8 9 14	15 25	26 30	31 32

CATEGORY	REPORT TYPE	REPORT SOURCE	DOCKET NUMBER	EVENT DATE	REPORT DATE
01 CONT	L	L	0 5 0 - 0 2 4 9	1 1 0 1 7 6	1 1 3 0 7 6
7 8 57 58	59 60	61 68	69 74	75 80	

EVENT DESCRIPTION

02 DURING REFUELING OUTAGE LOCAL LEAK-RATE TESTING, THE VOLUME

03 BOUNDED BY A BLIND FLANGE AND REACTOR HEAD COOLING INBOARD

04 ISOLATION CHECK VALVE 3-205-2-7 EXHIBITED A LEAK RATE OF

05 APPROXIMATELY 380 SCFH (TECH SPEC LIMIT: 29.381 SCFH), SINCE THE

06 BLIND FLANGE EXHIBITED NO DETECTABLE LEAKS, THE AS-FOUND

(SEE ATTACHED SHEET)

SYSTEM CODE	CAUSE CODE	COMPONENT CODE	PRIME COMPONENT SUPPLIER	COMPONENT MANUFACTURER	VIOLATION
07 C F	E	V A L V E X	N	M 3 6 0	Y
7 8 9 10	11	12 17	43	44 47	48

CAUSE DESCRIPTION

08 THE VALVE LEAKAGE WAS APPARENTLY CAUSED BY DIRT IN THE SEATING

09 AREA AND ON THE DISC PIVOT PIN. THE VALVE WAS CLEANED, AND THE

10 SEATING SURFACE WAS LAPPED. SUBSEQUENT RETESTING OF VALVE

(SEE ATTACHED SHEET)

FACILITY STATUS	% POWER	OTHER STATUS	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION
11 H	0 0 0	NA	B	NA
7 8 9	10 12 13	44	45	46

FORM OF ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY	LOCATION OF RELEASE
12 Z	Z	NA	NA
7 8 9	10 11	44	45

PERSONNEL EXPOSURES

NUMBER	TYPE	DESCRIPTION
13 0 0 0	Z	NA
7 8 9 11	12	13

PERSONNEL INJURIES

NUMBER	DESCRIPTION
14 0 0 0	NA
7 8 9 11	12

OFFSITE CONSEQUENCES

15 NA

LOSS OR DAMAGE TO FACILITY

TYPE	DESCRIPTION
16 Z	NA
7 8 9 10	

PUBLICITY

17 NA

ADDITIONAL FACTORS

18 NA

19

8306100295 761130
PDR ADOCK 05000249
S PDR

NAME: RANDOLPH C. WEIDNER

PHONE: EXT. 265

EVENT DESCRIPTION (Continued)

leakage was attributed entirely to leakage through check valve 3-205-2-7. A second leak-rate test of the volume bounded by the blind flange and redundant reactor head cooling outboard isolation valve M0 3-205-2-4 established that actual through-leakage for the penetration was 1.29 SCFH. Containment integrity, therefore, was not compromised by this failure. This is the first reported leak-rate test failure for this valve. (50-249/1976-31)

CAUSE DESCRIPTION (Continued)

3-205-2-7 yielded a leakage rate of 1.05 SCFH. Since check valve 205-2-7 has functioned satisfactorily over a period of several years, and since redundant valve 205-2-4 continues to perform satisfactorily, testing of this valve on an approximately annual basis should permit any recurring problem to be identified in a timely manner. Valve 3-205-2-7 is a 2½-inch Duo-Chek style B check valve, manufactured by the Mission Valve and Pump Co.



Commonwealth Edison
Dresden Nuclear Power Station
R.R. #1
Morris, Illinois 60450
Telephone 815/942-2920

IE FILE COPY

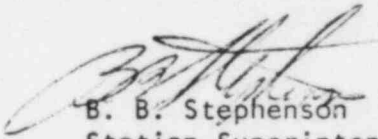
BBS Ltr. #76-834

November 30, 1976

Mr. James G. Keppler, Regional Director
Directorate of Regulatory Operations - Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137



Enclosed please find Reportable Occurrence report number 50-249/1976-31.
This report is being submitted to your office in accordance with the Dresden
Nuclear Power Station Technical Specifications, Section 6.6.B.


B. B. Stephenson
Station Superintendent
Dresden Nuclear Power Station

BBS:jo

Enclosure

cc: Director of Inspection & Enforcement
Director of Management Information & Program Control
File/NRC

12307