

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)
Turkey Point Plant - Unit 4

DOCKET NUMBER (2)

0 5 0 0 0 2 5 1

PAGE (3)

1 OF 0 1

TITLE (4)

Steam Generators Feedwater Nozzles Cracking

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)								
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)						
0	4	1	2	8	4	8	4	0	0	5	0	5	0	0	0	0	0
										N/A	0	5	0	0	0	0	0
										N/A	0	5	0	0	0	0	0

OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)									
POWER LEVEL (10)	0 1 0 1 0	20.402(b)	20.406(e)	50.73(a)(2)(iv)	73.71(b)						
		20.406(a)(1)(i)	50.36(a)(1)	50.73(a)(2)(v)	73.71(e)						
		20.406(a)(1)(ii)	50.36(a)(2)	50.73(a)(2)(vii)	<input checked="" type="checkbox"/> OTHER (Specify in Abstract below and in Text, NRC Form 365A) Voluntary Report						
		20.406(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)							
		20.406(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)							
		20.406(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(ix)							

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
Jesus Arias, Jr., Regulation and Compliance Lead Engineer	3 0 5 2 4 5 - 2 9 1 0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)	NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input type="checkbox"/>	<input checked="" type="checkbox"/>				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

During in-service inspection of the 4A steam generator feedwater nozzle to reducer weld area, crack-like ultrasonic indications were detected in the 18" x 14" reducer base metal. Subsequent to removal of the reducer on April 12, 1984, a circumferentially oriented crack was confirmed by liquid penetrant. The crack was found to be in the counterbore thickness transition approximately 270 degrees around the circumference.

Additional examinations performed on the other steam generators including the adjacent horizontal run of pipe revealed a similar condition on the 4C steam generator. This crack was found to be oriented identical to the one on 4A but with a length of approximately 180 degrees.

Repairs are being implemented (PC/M 84-80) by replacement of the reducers and elimination of internal thickness transitions. Under this PC/M, the feedwater nozzle extension is to be reduced in length (approximately 1/2 inch) to remove the counterbore.

The nozzle to reducer areas were examined on the 4B and Unit 3 3A, 3B, and 3C steam generators and no evidence of cracking was found.

The health and safety of the public were not affected. Similar occurrences: LER 251-80-08 and LER 250-80-19.

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PDR ADOCK 05000251
S PDR

May 14, 1984
PNS-LI-84-167

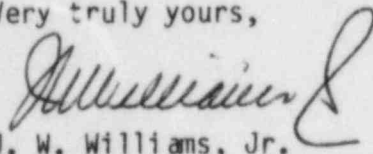
U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Gentlemen:

Re: Reportable Event 84-05
Turkey Point Unit 4
Date of Event: April 12, 1984
Voluntary Report
Steam Generator Feedwater Nozzles

The attached Licensee Event Report is being submitted as a voluntary report to provide notification of the subject event.

Very truly yours,


J. W. Williams, Jr.
Group Vice President
Nuclear Energy

JWW/PLP/js

Attachment

cc: J. P. O'Reilly, Region II, USNRC
Harold F. Reis, Esquire
File 933.1 TP

IE22
1/1