

SAXTON NUCLEAR EXPERIMENTAL CORPORATION
GENERAL PUBLIC UTILITIES SYSTEM

JERSEY CENTRAL POWER & LIGHT COMPANY
NEW JERSEY POWER & LIGHT COMPANY
PENNSYLVANIA ELECTRIC COMPANY
METROPOLITAN EDISON COMPANY

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Saxton, Pa.
December 9, 1971

Dr. Peter A. Morris, Director
Division of Reactor Licensing
U.S. Atomic Energy Commission
Washington, D.C. 20545

Dear Dr. Morris:

In accordance with Paragraph 3.E.(1)(b) of License DPR-4, Saxton herewith reports the accidental release of a small quantity of radioactive gases. This occurrence was reported by telephone to Mr. R. L. Spessard, Region I, Division of Compliance, on December 1, 1971.

The accidental release of radioactive gases occurred on November 29, 1971, while operating personnel were making a primary coolant boron dilution. While the dilution was in progress, radioactive gases leaked out of the purification system piping through the stem packing of a $\frac{1}{2}$ -inch valve and escaped into the charging pump room of the C&A Building. The gases were then picked up by the building ventilating system and exhausted to the environs via the plant stack.

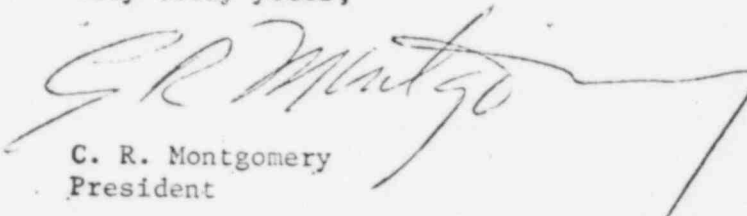
Using the stack monitor recorder (RIC-3) as a basis for estimating the activity released, we estimate 80.2 curies of gaseous activity were released to the environs over a 7-hour period. The maximum concentration of gaseous activity in the stack, excluding I-131, when averaged over a 15-minute period is estimated at 3.52×10^{-3} uCi/cc, which exceeds the 1×10^{-3} uCi/cc set forth in the Technical Specifications. The activity was predominately Xe-133 and Xe-135. The Iodine activity associated with this release was negligible. Based on the activity obtained from the stack gas Iodine sampler, we calculate 4.3 uCi of I-131 were released from the stack during the 7-day sample period, 11/24/71 - 12/1/71. The particulate activity associated with this release was also negligible. We calculate 0.75 uCi of particulate activity were released from the stack during the 7-day sample period, 11/24/71 - 12/1/71.

There were no over exposures to on-site personnel. For a release of 80.2 curies of gaseous activity from the stack and using a value of 2.3×10^{-4} sec/ m^3 for the short term atmospheric dilution factor and 1 Mev for \bar{E} we calculate the off-site whole body dose to be 4.23×10^{-3} rem.

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The above calculations confirm that this accidental release of radioactivity does not constitute a possible unsafe condition relative to the operation of the facility or to the health and safety of the public; nor does it involve an unreviewed safety question. We have reviewed the events and equipment associated with this release and have taken the necessary action to prevent a recurrence.

Very truly yours,



C. R. Montgomery
President

cc: G. F. Trowbridge, Esquire
R. L. Spessard, Region I,
Division of Compliance