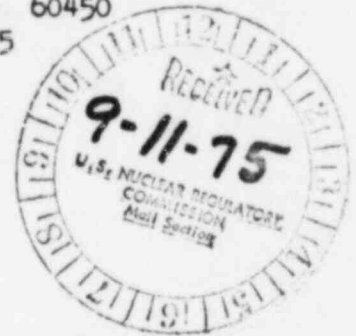




Commonwealth Edison
One First National Plaza, Chicago, Illinois
Address Reply to: Post Office Box 767
Chicago, Illinois 60690

BBS Ltr. #577-75

Dresden Nuclear Power Station
R. R. #1
Morris, Illinois 60450
September 3, 1975



Mr. James G. Keppler, Regional Director
Directorate of Regulatory Operation-Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137

SUBJECT: REPORT OF ABNORMAL OCCURRENCE PER SECTION 6.6.A OF THE TECHNICAL SPECIFICATIONS
MAIN STEAM LINE DRAIN VALVE MO 3-220-1 OVERLOAD RELAY FAILURE

- References: 1) Regulatory Guide 1.16 Rev 1 Appendix A
- 2) Notification of Region III of U. S. Nuclear Regulatory Commission
Telephone: R. Knop, 1710 hours on August 26, 1975
Telegram: J. Keppler, 1347 hours on August 27, 1975
- 3) Drawing Number M-345 and 12E-3505A

Report Number: 50-249/75-39

Report Date: September 3, 1975

Occurrence Date: August 26, 1975

Facility: Dresden Nuclear Power Station, Morris, Illinois

IDENTIFICATION OF OCCURRENCE

The overload relay tripped twice on motor-operated isolation valve 3-220-1, located inboard on the main steam drain line. This represents an abnormal occurrence because the malfunction of this valve could affect primary containment integrity.

CONDITIONS PRIOR TO OCCURRENCE

Unit-3 was in the shutdown mode for a refueling outage.

DESCRIPTION OF OCCURRENCE

At approximately 0900 hours on August 14, 1975, valve 220-1 was being functionally tested to evaluate changes which had been made to the torque switch settings. During the testing the valve tripped twice.

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September 3, 1975

DESIGNATION OF APPARENT CAUSE OF OCCURRENCE (Component Failure)

On August 26 an investigation revealed that the valve motor had tripped because of a defective overload relay. The relay appeared to be thermally damaged, but the cause of the damage is unknown. The motor current was measured and found to be within reasonable limits. The overload relay was apparently opening in response to normal current loads. The relay should not open unless the motor current becomes excessive.

ANALYSIS OF OCCURRENCE

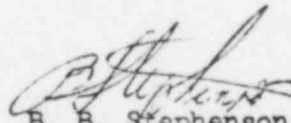
The malfunction of valve 220-1 had little safety significance. Had the valve tripped during reactor operation, the leakage past the outboard valve 220-2 would have been well within Technical Specification limits even in the event of a loss-of-coolant accident. Plant personnel and the public were not endangered by this occurrence.

CORRECTIVE ACTION

The overload relay was replaced, and the valve was cycled satisfactorily several times.

FAILURE DATA

The overload relay is a General Electric model CR 124-K028. This is the first time an overload relay has failed on this valve.


B. B. Stephenson
Superintendent

BBS:ELS:smp

File/NRC