

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) SURREY POWER STATION, UNIT NO. 1	DOCKET NUMBER (2) 0 5 0 0 0 2 8 0	PAGE (3) 1 OF 0 3
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TITLE (4)
Iodine Spike

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0	4	07	84	84	-	009	-	000	51084		0 5 0 0 0

OPERATING MODE (9) N		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)									
POWER LEVEL (10) 0 0 10	20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)							
	20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)							
	20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	<input checked="" type="checkbox"/> OTHER (Specify in Abstract below and in Text, NRC Form 366A)							
	20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	"SPECIAL REPORT"							
	20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)								
20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(ix)									

LICENSEE CONTACT FOR THIS LER (12)								TELEPHONE NUMBER			
J. L. Wilson - Station Manager								AREA CODE		8 0 4	
								3 5 7		- 3 1 8 4	

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)											
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM

SUPPLEMENTAL REPORT EXPECTED (14)								EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)								<input checked="" type="checkbox"/> NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On April 7, 1984, at 0500 hours following a unit shutdown from 100% power, the specific activity sample of the reactor coolant showed a peak dose equivalent I-131 level of 1.56 microcuries/cc. This exceeds the dose equivalent I-131 limit of ≤ 1.0 microcuries/cc specified in Tech. Specs. 3.1.D.2 and is being reported in accordance with the Special Reporting requirements outlined in T.S.-3.1.D.4.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
SURRY POWER STATION, Unit 1	0 5 0 0 0 2 8 0	8 4	— 0 0 9	— 0 0	0 2	OF	0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Description of the Event

On April 7, 1984, at 0500 hours following a unit shutdown from 100%, the Specific Activity Sample of the reactor coolant showed a peak dose equivalent I-131 level of 1.56 microcuries/-c. This exceeds the dose equivalent I-131 limit of ≤ 1.0 microcuries/cc specified in Tech. Spec. 3.1.D.2 and is being reported in accordance with the Special reporting requirements outlined in Tech. Spec. 3.1.D.4.

Probable Consequences and Status of Redundant Equipment

The limitations on the specific activity of the primary coolant ensure that the resulting 2 hour doses at the site boundary will not exceed an appropriately small fraction of 10 CFR 100 limits following a postulated steam generator tube rupture. Since the dose equivalent I-131 peak was below the Technical Specification upper limit of 10 microcuries/cc, the reactor coolant gross activity was below the value analyzed in the FSAR for a tube rupture and 1% failed fuel. Therefore, the health and safety of the public were not affected.

Cause

The Iodine Spike was caused by known, but not specifically located, fuel element defects in the reactor core. Post Shutdown conditions enhanced the release of fission products, specifically I-131. This caused an increase of the reactor coolant specific activity.

Immediate Corrective Action

The immediate corrective action was to implement the actions required by Tech. Spec. Table 4.1-2B. Specifically, the level of the dose equivalent I-131 was monitored at least once every 4 hours until the level returned to less than 1.0 microcuries/cc.

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SUPPLEMENTAL INFORMATION

The supplemental information required by T.S.3.1.D.4 "Special Report" is included as follows:

1. Reactor Power History 48 hours prior to the event:

April 5, 1984 - 48 hours at 100%
 April 6, 1984 - 24 hours at 100%
 April 7, 1984 @ 0500 - Unit at 0% following ramp.

2. Fuel burnup by core region - as of February 25, 1984.

Fuel Batch : S2/6B: 29960 MWD/MTU
 6C: 32816 MWD/MTU
 4C: 32512 MWD/MTU
 7A: 29332 MWD/MTU
 7B: 37703 MWD/MTU
 8A: 27221 MWD/MTU
 8B: 27492 MWD/MTU
 9: 9397 MWD/MTU
 Cycle 7 Burnup : 8216 MWD/MTU

3. Prior to the reactor shutdown, the unit had established a normal letdown rate of 106 gpm.

4. No De-gassing operations were performed.

5. Duration of I-131 Spike:

April 6, 1984 @ 0015 - Routine Sample - 1.22 microcuries/cc
 April 7, 1984 @ 0440 - Post Shutdown Sample - 1.31 microcuries/cc
 0620 - Post Shutdown Sample - 1.30 microcuries/cc
 0820 - Post Shutdown Sample - 1.56 microcuries/cc
 1020 - Post Shutdown Sample - 1.53 microcuries/cc
 1220 - Post Shutdown Sample - 1.33 microcuries/cc
 1420 - Post Shutdown Sample - 1.20 microcuries/cc
 1620 - Post Shutdown Sample - 1.26 microcuries/cc
 1705 - Post Shutdown Sample - .280 microcuries/cc

Duration approximately 12 hours 25 minutes.