

CONTROL BLOCK:

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 P A S E S 1 2 0 0 - 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4 5  
7 8 9 14 15 25 26 30 37 CAT 38

CONT

0 1 REPORT SOURCE L 6 0 5 0 0 0 3 8 7 7 0 4 2 4 8 3 8 0 5 2 4 8 3 9  
7 8 60 61 65 66 69 74 75 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10

0 2 During the pre-commercial operation outage, removal of ECCS, Div. II out from of  
0 3 service left only one subsystem each of Core Spray and LPCI available. When a leak-  
0 4 ing relief valve was discovered in the CS keep-fill system, keep-fill was isolated  
0 5 and the CS system was declared inoperable. The remaining operable LPCI subsystem  
0 6 was insufficient to fulfill Tech. Spec. 3.5.2 requirements. No challenges to low  
0 7 pressure ECCS occurred and no water hammer incidents resulted from the situation.  
0 8 Therefore, no adverse consequences resulted.

0 9 SYSTEM CODE S F 11 CAUSE CODE B 12 CAUSE SUBCODE A 13 COMPONENT CODE V A L V E X 14 COMP. SUBCODE X 15 VALVE SUBCODE B 16  
7 8 9 10 11 12 13 14 15 16  
17 LER/RO REPORT NUMBER 8 3 21 22 SEQUENTIAL REPORT NO. 0 6 9 23 24 OCCURRENCE CODE 0 3 25 26 REPORT TYPE L 27 28 REVISION NO. 0 29 30  
ACTION TAKEN E 18 FUTURE ACTION Z 19 EFFECT ON PLANT Z 20 SHUTDOWN METHOD Z 21 HOURS 0 0 0 0 22 ATTACHMENT SUBMITTED Y 23 NPRD-4 FORM SUB. N 24 PRIME COMP. SUPPLIER A 25 COMPONENT MANUFACTURER L 2 6 3 26  
33 34 35 36 37 40 41 42 43 44 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27

1 0 Operation with less than the required number of low pressure ECCS subsystem re-  
1 1 sulted from inadequate pressure control in the keep-fill system. To prevent lift-  
1 2 ing, efforts to bypass the PCV's and raise the relief valve setpoints above shut-  
1 3 off head were in progress when the incident occurred. These activities have been  
1 4 completed, resolving the problems with leaking keep-fill relief valves.

1 5 FACILITY STATUS B 28 % POWER 0 0 0 29 OTHER STATUS NA 30 METHOD OF DISCOVERY A 31 DISCOVERY DESCRIPTION Operator observation 32  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32  
1 6 ACTIVITY CONTENT Z 33 Z 34 AMOUNT OF ACTIVITY NA 35 LOCATION OF RELEASE NA 36  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32  
1 7 PERSONNEL EXPOSURES NUMBER 0 0 0 37 TYPE Z 38 DESCRIPTION NA 39  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32  
1 8 PERSONNEL INJURIES NUMBER 0 0 0 40 DESCRIPTION NA 41  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32  
1 9 LOSS OF OR DAMAGE TO FACILITY TYPE Z 42 DESCRIPTION NA 43  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32  
2 0 PUBLICITY ISSUED N 44 DESCRIPTION NA 45  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32

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PDR ADOCK 05000387  
S PDR

NRC USE ONLY

NAME OF PREPARER L.A. Kuczynski

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ATTACHMENT

LER # 83-069/03L-0

Pennsylvania Power & Light Company  
Susquehanna Steam Electric Station  
Docket Number: 50-387

Inability of pressure control valves to positively seat, coupled with relief valve settings below shutoff head of condensate storage and transfer (keep-fill) pumps resulted in frequent lifting of relief valves.

The ECCS divisional isolation was required to support RHRSW/ESW work necessary for Unit 1/Unit 2 tie-in (construction completion). The tie-in work has been completed and Division II ECCS returned to service.



# Pennsylvania Power & Light Company

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May 24, 1983

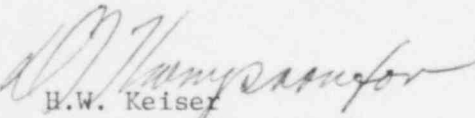
Mr. J.M. Allan  
Acting Regional Administrator, Region I  
U.S. Nuclear Regulatory Commission  
631 Park Avenue  
King of Prussia, PA 19406

SUSQUEHANNA STEAM ELECTRIC STATION  
LICENSEE EVENT REPORT 83-069/03L-0  
ER 100450 FILE 841-23  
PLA-1681

Docket No. 50-387  
License No. NPF-14

Dear Mr. Allan:

Attached please find a copy of Licensee Event Report No. 83-069/03L-0. This event was determined to be reportable per Technical Specification 6.9.1.9.b, in that during an outage with Division II Emergency Core Cooling System (ECCS) out of service, the Core Spray Pump Discharge Keep Fill System was isolated due to a leaking relief valve. This gave rise to the condition wherein there was only one operable low pressure ECCS loop available for service, less than the minimum requirements for the plant in Condition 5, per Technical Specification 3.5.2.



H.W. Keiser

Superintendent of Plant-Susquehanna

LAK/pjg

attachment

cc: Director of Nuclear Reactor Regulation  
Attention: Mr. A. Schwencer, Chief  
Licensing Branch No. 2  
Division of Licensing  
U.S. Nuclear Regulatory Commission  
Washington, DC 20555

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U.S. Nuclear Regulatory Commission  
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