

# AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-346  
 UNIT Davis-Besse Unit 1  
 DATE April 8, 1983  
 COMPLETED BY Bilal Sarsour  
 TELEPHONE 419-259-5000, Ext. 384

MONTH March, 1983

| DAY | AVERAGE DAILY POWER LEVEL<br>(MWe-Net) |
|-----|--|
| 1   | <u>876</u>                             |
| 2   | <u>875</u>                             |
| 3   | <u>879</u>                             |
| 4   | <u>866</u>                             |
| 5   | <u>872</u>                             |
| 6   | <u>700</u>                             |
| 7   | <u>877</u>                             |
| 8   | <u>869</u>                             |
| 9   | <u>879</u>                             |
| 10  | <u>877</u>                             |
| 11  | <u>878</u>                             |
| 12  | <u>878</u>                             |
| 13  | <u>878</u>                             |
| 14  | <u>877</u>                             |
| 15  | <u>878</u>                             |
| 16  | <u>878</u>                             |

| DAY | AVERAGE DAILY POWER LEVEL<br>(MWe-Net) |
|-----|--|
| 17  | <u>872</u>                             |
| 18  | <u>859</u>                             |
| 19  | <u>879</u>                             |
| 20  | <u>861</u>                             |
| 21  | <u>877</u>                             |
| 22  | <u>877</u>                             |
| 23  | <u>878</u>                             |
| 24  | <u>878</u>                             |
| 25  | <u>878</u>                             |
| 26  | <u>878</u>                             |
| 27  | <u>877</u>                             |
| 28  | <u>877</u>                             |
| 29  | <u>878</u>                             |
| 30  | <u>877</u>                             |
| 31  | <u>877</u>                             |

## INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

8306020129 8:30408  
 PDR ADOCK 05000346  
 R PDR

# OPERATING DATA REPORT

DOCKET NO. 50-346  
 DATE April 8, 1983  
 COMPLETED BY Bilal Sarsour  
 TELEPHONE 419-259-5000,  
 Ext. 384

## OPERATING STATUS

1. Unit Name: Davis-Besse Unit 1
2. Reporting Period: March, 1983
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 925
5. Design Electrical Rating (Net MWe): 906
6. Maximum Dependable Capacity (Gross MWe): 918
7. Maximum Dependable Capacity (Net MWe): 874
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

Notes

9. Power Level To Which Restricted, If Any (Net MWe): \_\_\_\_\_
10. Reasons For Restrictions, If Any: \_\_\_\_\_

|   | This Month       | Yr.-to-Date      | Cumulative        |
|---|------------------|------------------|-------------------|
| 11. Hours In Reporting Period               | <u>744.0</u>     | <u>2,160.0</u>   | <u>40,921.0</u>   |
| 12. Number Of Hours Reactor Was Critical    | <u>744.0</u>     | <u>1,833.3</u>   | <u>22,728.8</u>   |
| 13. Reactor Reserve Shutdown Hours          | <u>0.0</u>       | <u>313.9</u>     | <u>3,678.0</u>    |
| 14. Hours Generator On-Line                 | <u>744.0</u>     | <u>1,808.6</u>   | <u>21,568.2</u>   |
| 15. Unit Reserve Shutdown Hours             | <u>0.0</u>       | <u>0.0</u>       | <u>1,732.5</u>    |
| 16. Gross Thermal Energy Generated (MWH)    | <u>2,024,447</u> | <u>4,894,742</u> | <u>50,267,503</u> |
| 17. Gross Electrical Energy Generated (MWH) | <u>679,685</u>   | <u>1,643,711</u> | <u>16,749,365</u> |
| 18. Net Electrical Energy Generated (MWH)   | <u>647,067</u>   | <u>1,557,728</u> | <u>15,673,168</u> |
| 19. Unit Service Factor                     | <u>100.0</u>     | <u>83.7</u>      | <u>52.7</u>       |
| 20. Unit Availability Factor                | <u>100.0</u>     | <u>83.7</u>      | <u>56.9</u>       |
| 21. Unit Capacity Factor (Using MDC Net)    | <u>99.5</u>      | <u>82.5</u>      | <u>43.8</u>       |
| 22. Unit Capacity Factor (Using DER Net)    | <u>96.0</u>      | <u>79.6</u>      | <u>42.3</u>       |
| 23. Unit Forced Outage Rate                 | <u>0.0</u>       | <u>16.3</u>      | <u>20.3</u>       |

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):  
July 29, 1983      Refueling Outage      Duration: Approximately 8 weeks

25. If Shut Down At End Of Report Period, Estimated Date of Startup: \_\_\_\_\_
26. Units In Test Status (Prior to Commercial Operation):

INITIAL CRITICALITY  
 INITIAL ELECTRICITY  
 COMMERCIAL OPERATION

| Forecast | Achieved |
|----------|----------|
| _____    | _____    |
| _____    | _____    |
| _____    | _____    |

## UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH March, 1983DOCKET NO. 50-346UNIT NAME Davis-Besse Unit 1DATE April 8, 1983COMPLETED BY Bilal SarsourTELEPHONE 419-259-5000, Ext. 384

| No | Date     | Type <sup>1</sup> | Duration<br>(Hours) | Reason <sup>2</sup> | Method of<br>Shutting<br>Down Reactor <sup>3</sup> | Licensee<br>Event<br>Report # | System<br>Code <sup>4</sup> | Component<br>Code <sup>5</sup> | Cause & Corrective<br>Action to<br>Prevent Recurrence  |
|----|----------|-------------------|---------------------|---------------------|--|-------------------------------|-----------------------------|--------------------------------|--|
| 3  | 83 03 05 | F                 | 0                   | A                   | 5  | NP-33-83-18                   | RB                          | CKTBRK                         | Reactor power was reduced to approximately 55% by a plant runback when control rod drive group 7 control rod 12 ratchet tripped.<br><br>See Operational Summary for further details. |

<sup>1</sup>  
F: Forced  
S: Scheduled

<sup>2</sup>  
Reason:  
A-Equipment Failure (Explain)  
B-Maintenance or Test  
C-Refueling  
D-Regulatory Restriction  
E-Operator Training & License Examination  
F-Administrative  
G-Operational Error (Explain)  
H-Other (Explain)

<sup>3</sup>  
Method:  
1-Manual  
2-Manual Scram.  
3-Automatic Scram.  
4-Continuation from Previous Month  
5-Load Reduction  
9-Other (Explain)

<sup>4</sup>  
Exhibit G - Instructions  
for Preparation of Data  
Entry Sheets for Licensee  
Event Report (LER) File (NUREG-  
0161)

<sup>5</sup>  
Exhibit I - Same Source

OPERATIONAL SUMMARY  
MARCH, 1983

3/1/83 - 3/6/83: Reactor power was maintained at approximately 99% of full power until Control Rod Group 7 Rod 12 ratchet tripped into the core. A plant runback was initiated which reduced plant power to approximately 55%. The cause of the ratchet trip was determined to be a blown fuse on the "B" phase of a control rod drive motor power supply. The fuse was replaced, and Rod 7-12 repulled at 0145 hours.

Reactor power was restored to 86% by 0715 hours for valves testing. Reactor power slowly increased and attained approximately 99% by 2200 hours on March 6, 1983.

3/7/83 - 3/31/83: Reactor power was maintained at approximately 99% until 2000 hours on March 17, 1983, when power was reduced to approximately 96% to perform Surveillance Test ST 5010.02, "Moderator Temperature Coefficient Physics Testing". After the completion of ST 5010.02, reactor power increased steadily to approximately 99% and maintained at this level for the rest of the month.

REFUELING INFORMATION

DATE: March, 1983

1. Name of facility: Davis-Besse Unit 1
2. Scheduled date for next refueling shutdown: July 29, 1983
3. Scheduled date for restart following refueling: September 23, 1983
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? If answer is yes, what in general will these be? If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

Ans: Expect the Reload Report to require standard reload fuel design Technical Specification changes (3/4.1 Reactivity Control Systems and 3/4.2 Power Distribution Limits).

5. Scheduled date(s) for submitting proposed licensing action and supporting information: June, 1983
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

Ans: None identified to date.

7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.

(a) 177 (b) 92 - Spent Fuel Assemblies

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.

Present: 735 Increase size by: 0 (zero)

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

Date: 1993 - assuming ability to unload the entire core into the spent fuel pool is maintained.

COMPLETED FACILITY CHANGE REQUEST

FCR NO: 80-181

SYSTEM: Safety Features Actuation System (SFAS)

COMPONENT: Logic modules

CHANGE, TEST OR EXPERIMENT: On July 24, 1980, work implemented by FCR 80-181 was completed. This provided a delay in tripping of output logics, five per SFAS logic channel, that are on sequence steps other than sequence step 1.

REASON FOR CHANGE: This has improved sequencer operation by proper sequencing of loads on the diesel generator. This change will also allow the output logic to be blocked before tripping.

SAFETY EVALUATION: This FCR called for providing a time delay in tripping of the following logic modules in the SFAS:

| <u>CHANNEL 1</u> | <u>CHANNEL 2</u> | <u>CHANNEL 3</u> | <u>CHANNEL 4</u> | <u>SEQUENCE<br/>STEP</u> |
|------------------|------------------|------------------|------------------|--------------------------|
| L211             | L212             | L213             | L214             | 2                        |
| L221             | L222             | L223             | L224             | 5                        |
| L241             | L242             | L243             | L244             | 4                        |
| L311             | L312             | L313             | L314             | 3                        |
| L411             | L412             | L413             | L414             | 5                        |
| L511             | L512             | L513             | L514             | 5                        |

These logic modules are on sequence steps other than sequence step 1 and have to be blocked by the sequencer in case of a SFAS trip with a loss of offsite power.

During the analysis of results from ST 5031.07, Integrated SFAS Test, conducted in June 1980, it was discovered that several of the above logic modules tripped before their respective sequence step. Soon after their tripping, the tripped logic modules went to the untripped state. These logic modules tripped again, later, at their respective sequence steps, as designed. From this, it was concluded that the trip signals from these modules were not being timely blocked by the sequencer.

This FCR modified the output modules listed above so that they will be timely blocked, after a SFAS trip with loss of offsite power, until tripped by the sequencer. The time delay in tripping of these logic modules enables proper blocking from the sequencer.

The Final Safety Analysis Report (FSAR) Table 7-7 states that the SFAS response time should be less than or equal to five seconds for all measured variables. After adding the required equipment to provide the time delay, the field measurements have indicated that the delay ranges anywhere from 45 to 82 milliseconds. This time delay is negligible in comparison with the response time requirement provided in the FSAR. Moreover, response

time testing has indicated considerable margin implying that the FSAR and Technical Specification response times will still be met with the added delay.

Pursuant to the above, the change implemented by this FCR does not involve an unreviewed safety question.



COMPLETED FACILITY CHANGE REQUEST

FCR NO: 82-103

SYSTEM: Containment Air Sample System

COMPONENT: Valves HV5010E and HV5011E

CHANGE, TEST OR EXPERIMENT: This FCR was implemented to reduce the torque switch settings for Limitorque actuators for valves HV5010E and HV5011E. The new settings, both for opening and closing, are 1.0. Work was completed on August 11, 1982.

REASON FOR CHANGE: The original torque switch settings of these two valves caused the valve stems to be overtorqued and bent. The reduced settings will help prevent this.

SAFETY EVALUATION: This FCR called for decreasing the torque switch settings for containment air sample return valves HV5010E and HV5011E.

The safety function of these valves is to isolate containment on a Safety Features Actuation Incident Level 1.

The previous torque dial setting of 2.0 had resulted in the bending of the valve stem during valve operation.

The calculated values for the dial settings for both valves are 0.99 for opening and 0.785 for closing. It is convenient in the field to set the dials at 1.0.

The new settings of 1.0, both for closing and opening, have enhanced the equipment operation and are still sufficient to perform their function. Therefore, this change has not affected the safety function of HV5010E and HV5011E. Hence, no unreviewed safety question is involved.





April 8, 1980

Log No. K83-572  
File: RR 2 (P-6-83-03)

Docket No. 50-346  
License No. NPF-3

Mr. Norman Haller, Director  
Office of Management and Program Analysis  
U. S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Dear Mr. Haller:

Monthly Operating Report, March, 1983  
Davis-Besse Nuclear Power Station Unit 1

Enclosed are ten copies of the Monthly Operating Report for Davis-Besse Nuclear Power Station Unit 1 for the month of March, 1983.

Yours truly,

*Terry D. Murray / TDM*

Terry D. Murray  
Station Superintendent  
Davis-Besse Nuclear Power Station

TDM/BMS/ljk

Enclosures

cc: Mr. James G. Keppler  
Regional Administrator, Region III  
Encl: 1 copy

Mr. Richard DeYoung, Director  
Office of Inspection and Enforcement  
Encl: 2 copies

Mr. Tom Peebles  
NRC Resident Inspector  
Encl: 1 copy

*IE24*  
*1/1*