

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) LaSalle County Station Unit 2										DOCKET NUMBER (2) 0 5 0 0 0 3 7 4				PAGE (3) 1 OF 0 3		
TITLE (4) Reactor Manual Scram Due to Loss of Normal Feedwater																
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)						
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES None			DOCKET NUMBER(S) 0 5 0 0 0				
0 4	2 6	8 4	8 4	0 1 2	0 0 0	4	2 7	8 4				0 5 0 0 0				
OPERATING MODE (9) 2		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5 (Check one or more of the following) (11)														
POWER LEVEL (10) 0 0 1		20.402(b)				20.405(e)				<input checked="" type="checkbox"/> 50.73(a)(2)(iv)				73.71(b)		
		20.405(a)(1)(i)				50.36(a)(1)				50.73(a)(2)(v)				73.71(a)		
		20.405(a)(1)(ii)				50.36(a)(2)				50.73(a)(2)(vi)				OTHER (Specify in Abstract below and in Text, NRC Form 305A)		
		20.405(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(vii)(A)						
		20.405(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(vii)(B)						
		20.405(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(viii)						
LICENSEE CONTACT FOR THIS LER (12)																
NAME Tom Hammerich, extension 259										TELEPHONE NUMBER AREA CODE 8 1 5 3 5 7 - 6 7 6 1						
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC						
X	SID	ISITR	0 0 0 0	N												
X	SIG	ILR	0 0 4 0	N												
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)												<input checked="" type="checkbox"/> NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

At 0510 on April 26, 1984, Unit 2 was manually scrambled. Prior to the scram a shutdown was in progress. Reactor scram was performed as a precaution to prevent equipment damage. At the time of the event the main condenser hotwell level became excessively low due to a stuck level recorder while the level control system was operating in manual. Also the suction strainers for the condensate pumps became increasingly plugged due to placing the feedwater heaters on line earlier for the first time causing foreign materials to be carried over into the condenser. Therefore a potential loss of the Condensate System was possible. The affected strainers have been cleaned and the strainers and condensate polishers will be closely watched during subsequent feedwater heater evolutions. The level recorder was repaired and a special log of hotwell level initiated while level control is in manual.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/96

FACILITY NAME (1)

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LaSalle County Station Unit 2

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TEXT IF more space is required, use additional NRC Form 388A's (17)

I. EVENT DESCRIPTION

At 0510 on April 26, 1984, Unit 2 was manually scrammed. Prior to the scram RCIC (BN) was manually started in the Test Mode to ensure alternate water source to the reactor and as a means of pressure control. The Main Steam Isolation Valves (SB) were closed to maintain reactor pressure. RCIC was subsequently shut down at 0530 when reactor pressure was able to be controlled with losses to ambient.

Prior to the scram a reactor shutdown was in progress. At the time of the scram, the reactor was in Mode 2 (Startup) at approximately 450 psig.

The unit is in the Initial Startup Test Phase.

II. CAUSE

The reactor was scrammed manually as a precaution to prevent equipment damage. The suction strainers for the condensate pumps (SD) and the condensate polishers (SF) had become increasingly plugged over the previous several hours due to prior evolutions with placing feedwater heaters (SM) in service. Apparently this caused foreign material to be carried over into the Condensate System which caused condensate pump suction strainer and condensate polisher high differential pressures. At the time of the scram, potential loss of the Condensate System caused the operators to shut down the reactor quickly. Subsequently, it was determined that the main condenser (SG) hotwell level had dropped during the period contributing to loss of NPSH to the condensate pumps. This was due to a stuck recorder pen on the level recorder in the Control Room while the level control system was in manual control. (Auto was not operable.)

III. POTENTIAL CONSEQUENCES OF THE OCCURRENCE

At no time did the Condensate System fail to provide water to the reactor as needed. The operators began reducing power earlier when it became apparent that the Condensate System was having problems. All ECCS Systems and RCIC were available at the time. The RPS System (JC) functioned as expected.

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TEXT (If more space is required, use additional NRC Form 388A's) (17)

IV. CORRECTIVE ACTIONS

The level recorder was repaired and recalibrated. A special log of hotwell level was initiated while the level control is in the manual mode to ensure proper hotwell level is maintained. The affected Condensate System strainers have been cleaned. Sufficient condensate polishers have been cleaned prior to restart to ensure proper system operation.

During the initial startup phase of the feedwater heaters, a closer look will also be taken at trends on the condensate/condensate booster pump suction strainers and condensate polisher dP and flow to anticipate similar problems while starting up the feedwater heaters.

V. PREVIOUS OCCURRENCES

None.

VI. NAME AND TELEPHONE NUMBER OF PREPARER

Tom Hammerich, 815/357-6761, extension 259.



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May 1, 1984

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Dear Sir:

Reportable Occurrence Report #84-012-00, Docket #050-374 is being submitted to your office in accordance with 10 CFR 50.73.

G. J. Diederich
G. J. Diederich
Superintendent
LaSalle County Station

GJD/MLD/kg

Enclosure

xc: NRC, Regional Director
INPO-Records Center
File/NRC

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