



## Duquesne Light

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May 9, 1984

United States Nuclear Regulatory Commission  
Washington, DC 20555

ATTENTION: Mr. George W. Knighton, Chief  
Licensing Branch 3  
Office of Nuclear Reactor Regulation

SUBJECT: Beaver Valley Power Station - Unit No. 2  
Docket No. 50-412  
S.E.R. Outstanding Issues Status


Gentlemen:

Attachment 1 is a current list, as of May 1, 1984, which provides our understanding of outstanding issues identified in Table 1.2 of the draft Safety Evaluation Report. Items identified as "complete" are those for which responses have been provided and no confirmation of status has yet been received from the staff. We consider these items satisfactorily closed unless notified otherwise. In order to permit timely resolution of items identified as "complete" which may not be resolved to the staff's satisfaction, please provide a specific description of the issue which remains to be resolved.

Attachment 2 is a current list, as of May 1, 1984, which identifies draft SER sections not provided in the March 1, 1984, issue of the draft SER.

Items identified as "closed" or "confirmatory" are based upon formal or informal communications or agreements with the staff. Please inform us of any differing opinion of status described on the attached list.

DUQUESNE LIGHT COMPANY

By   
E. J. Woolever  
Vice President

GLB/wjs  
Attachment

cc: Mr. H. R. Denton, Director NRR (w/a)  
Mr. D. Eisenhut, Director Division of Licensing (w/a)  
Mr. T. Novak, Assistant Director Division of Licensing (w/a)  
Mr. G. Walton, NRC Resident Inspector (w/a)  
Mr. M. Lacitra, Project Manager (w/a)

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## ATTACHMENT 1

OUTSTANDING ISSUE	SUBJECT	PROPOSED RESPONSE DATE	STATUS	LETTER NO. AND DATE
1	Potential for flooding from local intense precipitation		1	
2	Flooding from Peggs Run		1	
3	Adequacy of the proposed tech spec for drought conditions	4/30/84	Complete	2NRC-4-049, 4/30/84
4	Adequacy of intake structure silt monitoring program	4/30/84	Complete	2NRC-4-049, 4/30/84
5	Boundaries of the Appalachian Plateau Tectonic Province	4/30/84	Complete	2NRC-4-048, 4/27/84
6	Maximum earthquake	4/30/84	2	2NRC-4-048, 4/27/84
7	Ground motion taking soil properties into consideration	4/30/84	2	2NRC-4-048, 4/27/84
8	Actual earthquake time histories	4/30/84	2	2NRC-4-048, 4/27/84
9	Shallow earthquakes	4/30/84	Complete	2NRC-4-048, 4/27/84
10	Vertical seismic design accelerations	4/30/84	2	2NRC-4-048, 4/27/84
11	Tornado pressure drop rate calculations	4/30/84	Complete	2NRC-4-047, 4/27/84
12	Concrete missile barrier design		Complete	2NRC-4-018, 2/27/84
13	Site-specific response spectra	4/30/84	2	2NRC-4-047, 4/27/84
14	Justification of peak spreading of floor response spectra		Complete	2NRC-4-018, 2/27/84
15	3-component seismic input vs. 2-component input	6/15/84		
16	Soil-structure interaction analysis of containment and intake structure	6/15/84		
17	Significance of the coupling in structural response between mutually orthogonal axes of structures	6/15/84		
18	Deviations of containment design from ASME Code Section III Division 2	6/15/84		
19	Ultimate capacity analysis of containment	6/15/84		
20	Deviations from requirements of ACI 349 code as augmented by Reg. Guide 1.142	4/27/84	Complete	2NRC-4-047, 4/27/84
21	Structural audit action items	4/27/84	Complete	2NRC-4-047, 4/27/84
22	Affects of cracked panel on dynamic modeling and design		Complete	2NRC-4-018, 2/27/84
23	Adequacy of re-assessed safety factors against sliding and overturning of containment, aux. building, and intake structure	4/30/84	Complete	2NRC-4-047, 4/27/84
24	Through-wall leakage cracks in moderate energy lines in containment	5/15/84		
25	Selection of postulated break locations	5/15/84		
26	Jet impingement effects	6/29/84		

OUTSTANDING ISSUE	SUBJECT	PROPOSED RESPONSE DATE	STATUS	LETTER NO. AND DATE
27	Break exclusion zones	5/15/84		
28	Pipe-to-pipe impact	5/15/84		
29	Limited break areas	5/15/84		
30	Saturated or subcooled water blowdown	5/15/84		
31	Design of pipe rupture restraints	5/15/84		
32	List of transients	5/15/84		
33	Location of systems to be monitored during pre-op testing	6/01/84		
34	Criteria used for determining acceptability of vibration levels	5/04/84		
35	Vibration monitoring program	5/04/84		
36	Combining 3 components of earthquake motion	5/04/84		
37	Modes included in seismic analyses	5/15/84		
38	Demonstrating seismic qualification	5/15/84		
39	Loading combinations, system operating transients, and stress limits	5/15/84		
40	HVAC system design	5/15/84		
41	Design of safety and relief valves	5/15/84		
42	Design and construction of ASME Class 1, 2, and 3 component support	5/15/84		
43	Preservice and inservice testing of pumps and valves	a-6/84 b-12/84 c-6/85		
44	Effects of rod bow on DNB	6/29/84		
45	Flow measurement capability and procedure	6/29/84		
46	Loose parts detection program report	6/29/84		
47	Documentation of complete ICC system	6/29/84		
48	Preservice inspections program	a-6/84		
49	Compliance with Appendix G 10CFR Part 50	6/29/84	Complete	2NRC-4-049, 4/30/84
50	Compliance with Appendix H 10CFR Part 50	4/30/84	Complete	2NRC-4-049, 4/30/84
51	Pressure temperature limits	4/30/84	Complete	2NRC-4-049, 4/30/84
52			3	
53	Control room habitability	6/01/84		
54	Inservice inspection of Class 2 and 3 components	a-6/84 c-6/85		
55	Design modification for automatic reactor trip		Complete	2NRC-4-033, 3/30/84

OUTSTANDING ISSUE	SUBJECT	PROPOSED RESPONSE DATE	STATUS	LETTER NO. AND DATE
56	Anticipatory reactor trip on turbine trip	5/25/84	Complete	2NRC-4-038, 4/10/84
57	P-4 interlock			
58	Undetectable failure in online testing circuitry for engineered safeguards relays	6/29/84		
59	Service water system isolation on low header pressure	5/31/84		
60	Normal letdown line relief valve	5/25/84	Complete	2NRC-4-032, 3/28/84
61	Switchover from injection to recirculation	5/11/84		
62	Main feedwater isolation	5/25/84		
63	Control room isolation	6/29/84		
64	Steam generator level control and protection		Complete	2NRC-4-032, 3/28/84
65	IE Bulletin 80-06 concerns	6/29/84		
66	Independence between manual and automatic action	6/29/84		
67	Power lockout for motor-operated valves			
68	Remote shutdown capability	5/25/84	5	
69	Emergency response capability			
70	Direct indication of relief and safety valve positions	5/18/84		
71	Bypass and inoperable status panel	5/25/84		
72	IE Bulletin 79-27	5/25/84	Complete	2NRC-4-040, 4/30/84
73	Reactor coolant system loop isolation interlocks	6/29/84		
74	Primary component cooling water isolation from RCP thermal barriers	6/29/84		
75	PID controller modification	6/29/84		
76	High energy line breaks and consequential control system failures	4/27/84	Complete	2NRC-4-042, 4/18/84
77	Control system failure caused by malfunctions of common power source or instrument line	5/25/84		
78	Procedure to estimate extent of core damage			
79	Backup post accident sampling through grab samples for inline analyses			
80	Measuring radionuclide concentrations		Complete	2NRC-4-042, 4/18/84
81	Performance of PASS instrumentation and analytical procedures			
82	Fire hazards analysis	5/15/84		
83	Fire brigade	5/15/84		
84	Penetration seals	5/15/84		

OUTSTANDING ISSUE	SUBJECT	PROPOSED RESPONSE DATE	STATUS	LETTER NO. AND DATE
85	Safe shutdown		5	
86	Alternate shutdown		5	
87	Hydrogen piping	5/15/84		
88	Cable tray suppression	5/15/84		
89	Power supplies for control room ventilation	5/15/84		
90	Fire detection	5/15/84		
91	Valve supervision	5/15/84		
92	Reactor coolant pumps and separation of safety-related components in containment	5/15/84		
93	Control room complex	5/15/84		
94	Cable spreading room	5/15/84		
95	Exemption to 10CFR 70.24(a) and description of alternative to required criticality monitors	6/01/84		
96	Types, numbers, specifications for portable and laboratory HP instruments	5/15/84		
97	Training/retraining for health physics professionals and verification that contractor training meets 10CFR 19.12 requirements		6	
98	Procedures generation package/TMI I.C.1, short-term accident analysis and procedures revision		7	
99	Physical security	5/16/84		
100	Initial test program	5/30/84		
101	Radiological consequences of control rod ejection DBA		5	
102	Radiological consequences of a small line break DBA	5/15/84		
103	Radiological consequences of a steam generator tube rupture DBA and review of operator actions and system performance	6/29/84		
104	Radiological consequences of a steamline break outside secondary containment		5	
105	Radiological consequences of a loss of coolant DBA		5	
106	Radiological consequences of a fuel handling DBA		5	
107			3	
108			3	
109			3	
110			3	

<u>OUTSTANDING ISSUE</u>	<u>SUBJECT</u>	<u>PROPOSED RESPONSE DATE</u>	<u>STATUS</u>	<u>LETTER NO. AND DATE</u>
111			3	
112	Technical specifications		8	
113	Detailed control room design review	6/01/85		
114	Safety parameter display system	8/01/84		

<sup>1</sup>Currently evaluating letter from T. M. Novak to E. J. Woolever dated April 11, 1984

<sup>2</sup>Supplemental submittal to be provided.

<sup>3</sup>Not provided in draft SER

<sup>4</sup>Further resolution being discussed informally

<sup>5</sup>Staff action only

<sup>6</sup>To be discussed at RAB meeting 5/01/84

<sup>7</sup>No response required

<sup>8</sup>DLC is currently arranging a management meeting with NRC to discuss this item

<sup>a</sup>PSI PROGRAM - 6/84

<sup>b</sup>PSI PROCEDURES FOR VALVES & PUMPS - 12/84

<sup>c</sup>ISI PROGRAM - 6/85

# ATTACHMENT 2

## Draft SER Sections and Dates Provided

<u>Section</u>	<u>Date</u>	<u>Section</u>	<u>Date</u>	<u>Section</u>	<u>Date</u>	<u>Section</u>	<u>Date</u>	<u>Section</u>	<u>Date</u>
2.1.1		5.2.3		8.4.4		10.4.4		15.4.5	
2.1.2		5.2.5		8.4.5		10.4.5		15.4.6	
2.1.3		5.4.1		8.4.6		10.4.7		15.5.1	
2.1.4		5.4.7		8.4.7		10.4.9		15.5.2	
2.2.1		5.4.11		8.4.8		11.1.1		15.7.1	
2.2.2		6.2.1		9.1.1		11.1.2		15.7.2	
2.2.3		6.2.2		9.1.4		11.2.1		15.7.3	
2.3.1		6.2.3		9.1.5		11.2.2		15.9.1	
2.3.2		6.2.4		9.2.1		11.3.1		15.9.2	
2.3.3		6.2.5		9.2.2		11.3.2		15.9.3	
2.3.4		6.2.6		9.2.3		11.4.1		15.9.4	
2.3.5		6.3.1		9.2.4		11.4.2		15.9.5	
2.4.13		6.3.2		9.2.5		11.5.1		15.9.6	
2.5.4		6.3.3		9.2.6		11.5.2		15.9.7	
2.5.5		6.3.4		9.3.1		13.1.1		15.9.8	
2.5.6		6.3.5		9.3.3		13.1.2		15.9.9	
3.2.1		6.3.6		9.4.1		13.3		15.9.10	
3.2.2		6.5.1		9.4.2		13.4		15.9.11	
3.4.1		6.5.3		9.4.3		13.5.1		15.9.12	
3.5.1		6.5.4		9.4.4		15.1.2		15.9.13	
3.5.2		7.2.3		9.4.5		15.1.3		15.9.14	
3.6.1		7.3.4		9.5.2		15.1.4		17.1	
3.10.1		7.4.3		9.5.3		15.1.5		17.2	
3.10.2		7.5.3		9.5.4		15.2.1		17.3	
3.11.1		7.6.3		9.5.5		15.2.2		17.4	
3.11.2		7.7.3		9.5.6		15.2.3		19	
3.11.3		8.2.1		9.5.7		15.2.4		20	
4.2.1		8.2.2		9.5.8		15.2.5		21	
4.2.2		8.2.3		10.2.3		15.2.6		22.1	
4.2.3		8.2.4		10.3.1		15.2.7		22.2	
4.2.4		8.3.1		10.3.2		15.2.8		22.3	
4.2.5		8.3.2		10.3.6		15.3.1		23	
4.5.1		8.4.1		10.4.1		15.3.2			
4.5.2		8.4.2		10.4.2		15.3.3			
5.2.1		8.4.3		10.4.3		15.3.4			