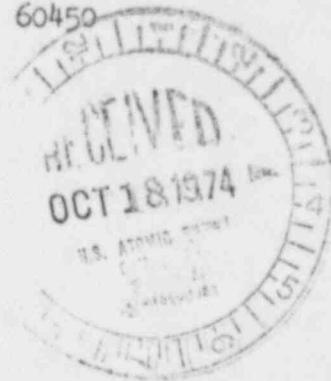




Commonwealth Edison
One First National Plaza, Chicago, Illinois
Address Reply to: Post Office Box 767
Chicago, Illinois 60690

BBS Ltr.#737-74

Dresden Nuclear Power Station
R. R. #1
Morris, Illinois 60450
October 10, 1974



Mr. James G. Keppler, Regional Director
Directorate of Regulatory Operations-Region III
U. S. Atomic Energy Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137

SUBJECT: REPORT OF ABNORMAL OCCURRENCE PER SECTION 6.6.A OF THE TECHNICAL SPECIFICATIONS.
PREMATURE ACTUATION OF UNIT 2 SAFETY VALVE 2-203-4E DURING HYDROSTATIC TEST.

- References: 1) Regulatory Guide 1.16 Rev.1 Appendix A
- 2) Notification of Region III of AEC Regulatory Operations
Telephone: Mr. P. Maura, 1320 hours on October 4, 1974
Telegram: Mr. J. Keppler, 1400 hours on October 4, 1974
- 3) Drawing Number: P&ID M-12

Report Number: 50-237/1974-50

Report Date: October 10, 1974

Occurrence Date: October 4, 1974

Facility: Dresden Nuclear Power Station, Morris, Illinois

IDENTIFICATION OF OCCURRENCE

At 0300 hours on October 4, 1974 safety valve 2-203-4E actuated at 1085 psig pressure which is less than the limit of $1210 \pm 1\%$ as specified by the Technical Specifications.

CONDITIONS PRIOR TO OCCURRENCE

At 0300 hours on October 4, 1974, Unit 2 was in the shutdown mode.

DESCRIPTION OF OCCURRENCE

At 0300 hours on October 4, 1974, a post maintenance hydrostatic test was in progress to test the repair to 2A recirc pump discharge bypass piping. Safety valve 2-203-4E actuated at 1085 psig, which is less than the limit of $1210 \pm 1\%$ as specified by the Technical Specifications.

*st-237
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COPY SENT REGION III

October 10, 1974

DESIGNATION OF APPARENT CAUSE OF OCCURRENCE (Component Failure)

The safety valve was tested on October 8, 1974 using the safety valve test facility. The pressure setpoint was found to be 1095 psig. The valve had been installed in the unit on October 2, 1972 after being pressure set using nitrogen.

Valves tested during the 1974 refueling outage for Unit 3 have shown that a valve cold pressure set using nitrogen will often lift at a pressure lower than the extrapolated nitrogen set. This coupled with the long operating time since the safety valve was last tested is believed to be the cause of the problem.

ANALYSIS OF OCCURRENCE

There were no safety consequences to the public or plant personnel as a result of this occurrence because safety valve operability is not required when the reactor is below 320°F. Only minor contamination resulted from the actuation and this was contained completely within the drywell. During operation, the reactor would be shut down by the high pressure scram before 1095 psig would be reached and the only consequences would be contamination released to the drywell.

CORRECTIVE ACTION

Safety valve 2-203-4E was removed and replaced with a valve that had been set $1210 \pm 1\%$ and confirmed on the safety valve test facility. In addition to this, all remaining safety valves will be tested on the safety valve test facility during the next refueling outage scheduled to begin in November 1974.

FAILURE DATA

Premature actuation of safety valves during operation has occurred 3 times in the past. The valves lifted in conjunction with a relief valve opening. The shock wave associated with the relief valve opening is thought to be partially responsible for the safety valve actuation. Premature actuation is believed to be the result of both the relief valve shock wave and the use of the nitrogen cold set.

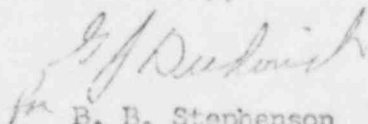
Valve Data

MFG Dresser

Type 6"-3777QA-RT-21

Size 6"

Sincerely,


B. B. Stephenson
Superintendent

BBS:do