

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Virgil C. Summer Nuclear Station										DOCKET NUMBER (2) 0 5 0 0 0 3 9 5 1 OF 0 1										PAGE 13			
TITLE (4) Pressurizer Safety Relief Valves																							
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)													
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)										
0	4	0	8	8	4	0	1	9	0	0	0	5	0	7	8	4	0	5	0	0	0	0	0
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 8: (Check one or more of the following) (11)																					
5		20.402(b)				20.406(e)				80.73(a)(2)(iv)				73.71(b)									
POWER LEVEL (10)		20.406(a)(1)(i)				80.36(a)(1)				80.73(a)(2)(v)				73.71(c)									
0		20.406(a)(1)(ii)				80.36(a)(2)				80.73(a)(2)(vi)				OTHER (Specify in Abstract below and in Text, NRC Form 355A)									
0		20.406(a)(1)(iii)				80.73(a)(2)(i)				80.73(a)(2)(viii)(A)				VOLUNTARY REPORT									
0		20.406(a)(1)(iv)				80.73(a)(2)(ii)				80.73(a)(2)(viii)(B)													
		20.406(a)(1)(v)				80.73(a)(2)(iii)				80.73(a)(2)(ix)													
LICENSEE CONTACT FOR THIS LER (12)																							
NAME A. R. Koon, Jr., Asso. Mgr., Regulatory Compliance														TELEPHONE NUMBER AREA CODE 8 0 3 3 4 5 - 5 2 0 9									
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																							
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS													
X	A B	R V	C 7 1 0	Y																			
SUPPLEMENTAL REPORT EXPECTED (14)														EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR					
YES (If yes, complete EXPECTED SUBMISSION DATE)														XXXX									

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

This Voluntary Report is being submitted as a follow-up to a telephone conference with NRC Region II on April 11, 1984.

During a scheduled outage which started March 23, 1984, Pressurizer Safety Relief Valve (PSRV) XVR-8010-C was removed and tested in accordance with the Inservice Inspection (ISI) Program. The valve lifted at 2590 psi, which was approximately 3.2% above Technical Specification setpoint ( $2485 \pm 1\%$ ). The valve was declared inoperable and, in accordance with the ISI Program, XVR-8010-B was tested satisfactorily with a relief setpoint of 2480 psi. XVR-8010-C was adjusted within the Technical Specification tolerance and reinstalled.

During our spring 1984 outage, XVR-8010-A was removed because of suspected leakage. The leakage was verified and the valve was replaced with an acceptable spare relief valve.

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PDR ADOCK 05000395  
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SOUTH CAROLINA ELECTRIC & GAS COMPANY

POST OFFICE 764

COLUMBIA, SOUTH CAROLINA 29218

O. W. DIXON, JR.  
VICE PRESIDENT  
NUCLEAR OPERATIONS

May 7, 1984

U.S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555

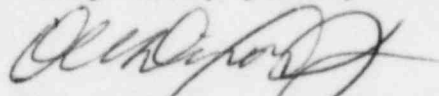
SUBJECT: Virgil C. Summer Nuclear Station  
Docket No. 50/395  
Operating License No. NPF-12  
LER 84-019

Dear Sir:

Please find attached Licensee Event Report #84-019 (Voluntary Report) for the Virgil C. Summer Nuclear Station.

Should there be any questions, please call us at your convenience.

Very truly yours,



O. W. Dixon, Jr.

HCF:RJB:OWD/dwf  
Attachment

cc: V. C. Summer  
T. C. Nichols, Jr./O. W. Dixon, Jr.  
E. H. Crews, Jr.  
E. C. Roberts  
W. A. Williams, Jr.  
D. A. Nauman  
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Group Managers  
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J. F. Heilman  
C. L. Ligon (NSRC)  
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