

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Brunswick Steam Electric Plant Unit 2										DOCKET NUMBER (2) 0 5 0 0 0 3 2 4										PAGE (3) 1 OF 0 1																																																													
TITLE (4) Failure To Meet Technical Specifications Action Statement 3.6.3a																																																																																	
EVENT DATE (5)										LER NUMBER (6)										REPORT DATE (7)										OTHER FACILITIES INVOLVED (8)																																																			
MONTH			DAY			YEAR				YEAR			SEQUENTIAL NUMBER			REVISION NUMBER				MONTH			DAY			YEAR				FACILITY NAMES										DOCKET NUMBER(S)																																									
0			2			28				8			4			8			4			-			0			0			5				-			0			0			5				0			7			8				4														0 5 0 0 0									
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OPERATING MODE (9) 1										THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)																																																																							
POWER LEVEL (10) 0 9 7										20.402(b)										20.406(c)										50.73(a)(2)(iv)										73.71(b)																																									
										20.406(a)(1)(i)										50.36(e)(1)										50.73(a)(2)(v)										73.71(e)																																									
										20.406(a)(1)(ii)										50.36(e)(2)										50.73(a)(2)(vii)										OTHER (Specify in Abstract below and in Text, NRC Form 365A)																																									
										20.406(a)(1)(iii)										X 50.73(a)(2)(i)										50.73(a)(2)(viii)(A)																																																			
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20.406(a)(1)(v)										50.73(a)(2)(iii)										50.73(a)(2)(x)																																																													
LICENSEE CONTACT FOR THIS LER (12)																																																																																	
NAME M. J. Pastva, Jr., Regulatory Compliance Technician																				TELEPHONE NUMBER 9 1 9 4 5 7 - 9 5 2 1																																																													
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																																																																																	
CAUSE			SYSTEM			COMPONENT			MANUFACTURER			REPORTABLE TO NRC			CAUSE			SYSTEM			COMPONENT			MANUFACTURER			REPORTABLE TO NRC																																																						
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SUPPLEMENTAL REPORT EXPECTED (14)																				EXPECTED SUBMISSION DATE (15)										MONTH DAY YEAR																																																			
YES (If yes, complete EXPECTED SUBMISSION DATE)																				X NO																																																													
ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)																																																																																	
<p>On 3-8-84, a review of plant documentation revealed that Technical Specifications (T/S) action statement 3.6.3a, which concerns the inoperability of primary containment isolation valves (PCIVs), was not met when the valve packing of the Unit 2 Reactor Water Cleanup (RWC) System inlet outboard isolation valve, 2-G31-F004, was adjusted on 2-28-84. On 1-28-84, when the subject maintenance was authorized, senior licensed personnel failed to follow plant instructions and reflect on the involved maintenance documents the necessity to enter a limiting condition for operation (LCO) when the subject maintenance was performed. Also, the involved documents were not adequately reviewed, resulting in required postmaintenance testing (PMT) of F004 not being performed prior to returning the valve to service. From 2-28-84 through 3-8-84, Unit 2 was operating at 97% power. F004 was tested for closure time and found satisfactory. The respective RWC System inboard isolation valve, 2-G31-F001, was operable and would have isolated the RWC System inlet if required. Further review revealed that four additional PCIVs (1-E11-F024A, 1-E11-F011B, 1-E11-F007B, and 2-E11-F027B) not required by Technical Specification 3.6.3 were also not properly tested following maintenance. These valves were tested and found satisfactory. By 7-31-84, appropriate personnel will receive applicable training on plant instructions dealing with LCO and PMT requirements. Instructions were issued to emphasize the need to appropriately enter an LCO when maintenance is done on T/S related valves and to coordinate such maintenance through the plant Operations Engineer.</p>																																																																																	
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Carolina Power & Light Company

Brunswick Steam Electric Plant

P. O. Box 10429

Southport, NC 28461-0429

May 7, 1984

FILE: B09-13516C

SERIAL: BSEP/84-1117

NRG Document Control Desk

U.S. Nuclear Regulatory Commission

Washington, DC 20555

BRUNSWICK STEAM ELECTRIC PLANT, UNIT 2

DOCKET NO. 50-324

LICENSE NO. DPR-62

LICENSEE EVENT REPORT 2-84-5

Gentlemen:

In accordance with Title 10 to the Code of Federal Regulations, the enclosed Licensee Event Report is submitted. In a letter dated April 5, 1984, Serial: BSEP/84-0939, it was conveyed that this event would be reported by May 7, 1984. This report is in accordance with the format set forth in NUREG-1022, September 1983.

Very truly yours,

C. R. Diecz, General Manager  
Brunswick Steam Electric Plant

RMP/jlh/LETJH1

Enclosure

cc: Mr. J. P. O'Reilly

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