

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Pilgrim Nuclear Power Station (PNPS) - Unit 1 DOCKET NUMBER (2) 0 5 0 0 0 2 9 3 1 OF 0 2

TITLE (4) Target Rock Safety Relief Valve Operability Problems

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0	4	0	4	8	4	0	0	5	0	0	0
0	4	0	4	8	4	0	0	5	0	0	0

OPERATING MODE (9)	POWER LEVEL (10)	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §. (Check one or more of the following) (11)									
N	0 0 0	20.402(b)	20.406(c)	50.73(a)(2)(iv)	73.71(b)						
		20.406(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)						
		20.406(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)						
		20.406(a)(1)(iii)	X 50.73(a)(2)(i)	50.73(a)(2)(viii)(A)							
		20.406(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)							
		20.406(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)							

LICENSEE CONTACT FOR THIS LER (12)

NAME Paul J. Hamilton - Plant Engineer TELEPHONE NUMBER 6 1 7 7 4 6 1 - 7 9 0 0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
X	S/B	R/V	T/O 2/O	Y					

SUPPLEMENTAL REPORT EXPECTED (14)

X YES (If yes, complete EXPECTED SUBMISSION DATE) NO EXPECTED SUBMISSION DATE (15) 0 9 2 1 8 4

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 4/4/84, during a refueling outage, the Maintenance Department was notified by Wyle Laboratories that the pilot valves on two of the Target Rock two-stage Safety Relief Valves (S/RV's) did not lift within specification when diagnostically tested in the as-found condition. This is contrary to the requirements of the intent of PNPS Technical Specification (T.S.) 2.2.B which required the S/RV's to lift at 1095 psi \pm 11 psi.

The most probable cause of the Safety Relief Valves not lifting has initially been determined to be stuck pilot valves.

Determination of root cause and corrective action is pending further analysis and testing.

8405080272 840504
PDR ADOCK 05000293
S PDRIE22
1/1

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Pilgrim Nuclear Power Station - Unit 1	0500029384	—	005	—00	02	OF	02

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On 4/4/84, during a refueling outage, the Maintenance Department was notified by Wyle Lab. that the pilot valves on two of the Target Rock two-stage Safety Relief Valves, Model No. 7567F, did not lift within specification when diagnostically tested in the "as-found" condition. The valves were being tested in accordance with the requirements of T.S. 4.6.D.1, 4.6.D.2, and 4.6.D.5. Section 2.2.B of T.S. required the S/RV's to lift at 1095 psi \pm 11 psi.

With the air operator actuated, the pilot valve disc on RV 203-3D, Serial No. 1049, did not lift with 200 psig Nitrogen pressure applied to the steam inlet connection. The valve was then disassembled in an attempt to determine cause of the pilot valve not lifting. Cause was determined to be that the pilot valve stuck on its seat. During inspection, a very small amount of material was noted on the disc and seat. Both surfaces were cleaned, and the valve refurbished and successfully tested.

The other pilot valve disc on RV 203-3A, Serial No. 1054, also did not lift with 200 psig Nitrogen applied to the steam inlet connection. This valve will be tested in early May 1984 to determine the actual lift point of the pilot valve. Subsequent to this test, the valve top works is planned to be sent to the Massachusetts Institute of Technology for metallurgical analysis and determination of root cause.

The main steam system design utilizes four of the two-stage Target Rock Relief Valves. The other two valves, RV 203-3B and RV 203-3C, did lift within specification when tested.

These Target Rock Valves are of the same design as those addressed in I&E Information Notices 83-39, 82-41, and General Electric Service Information Letter (SIL) No. 196 and associated supplements.

Determination of root cause and development of a corrective action plan will be provided in an update report. A task force has been established to address the issue.

A previous event of a similar nature was addressed in LER 81-062/01T-0 and update LER 81-062/01T-1.

This event did not impact the health and safety of the public.

BOSTON EDISON COMPANY
800 BOYLSTON STREET
BOSTON, MASSACHUSETTS 02199

WILLIAM D. HARRINGTON
SENIOR VICE PRESIDENT
NUCLEAR

May 4, 1984
BECO Ltr. #84-064

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Docket Number 50-293
License DPR-35

Dear Sir:

The attached Licensee Event Report #84-005, "Target Rock Safety Relief Valve Operability Problems," is hereby submitted in accordance with the requirements of 10CFR50.73.

If there are any questions on this subject, please do not hesitate to contact me.

Respectfully submitted,

W D Harrington
W. D. Harrington

PH:caw

Enclosure: LER 84-005-00

cc: Dr. Thomas E. Murley
Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

Standard BECO LER Distribution

IE22
1/1