

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)
Joseph M. Farley - Unit 2

DOCKET NUMBER (2)

0 5 0 0 0 3 6 4 1 OF 0 2

PAGE (3)

TITLE (4)

Reactor Trip

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0	4	0	9	8	4	8	4	0	0	5	0
0	4	0	9	8	4	0	0	5	0	4	8
0	4	0	9	8	4	0	0	5	0	4	8

OPERATING MODE (9)

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §. (Check one or more of the following) (11)

POWER LEVEL (10) 1 1 0 0	20.402(b)	20.406(e)	<input checked="" type="checkbox"/>	80.73(a)(2)(iv)	73.71(b)
	20.406(a)(1)(i)	80.36(e)(1)	<input type="checkbox"/>	80.73(a)(2)(v)	73.71(e)
	20.406(a)(1)(ii)	80.36(e)(2)	<input type="checkbox"/>	80.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
	20.406(a)(1)(iii)	80.73(a)(2)(i)	<input type="checkbox"/>	80.73(a)(2)(viii)(A)	
	20.406(a)(1)(iv)	80.73(a)(2)(ii)	<input type="checkbox"/>	80.73(a)(2)(viii)(B)	
	20.406(a)(1)(v)	80.73(a)(2)(iii)	<input type="checkbox"/>	80.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12)

NAME

TELEPHONE NUMBER

W. G. Hairston, III

AREA CODE

2 0 5 8 9 9 - 5 1 5 6

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS

SUPPLEMENTAL REPORT EXPECTED (14)

EXPECTED SUBMISSION DATE (15)

MONTH DAY YEAR

YES (If yes, complete EXPECTED SUBMISSION DATE)

☒ NO

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

At 1107 on 4-9-84, the reactor tripped due to Lo Level in 2A steam generator in conjunction with Loop 1 Steam Flow Greater Than Feed Flow. This was caused by operator error during performance of FNP-2-STP-33.1 (Safeguards Test Cabinet Train A(B) Functional Test). Health/safety of the public was not affected.

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PDR ADOCK 05000348
S PDR

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1) Joseph M. Farley - Unit 2	DOCKET NUMBER (2) 0 5 0 0 0 3 6 4 8 4 - 0 0 5 - 0 0 0 2 OF 0 2				LER NUMBER (6)			PAGE (3)		
					YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On 4-9-84, a plant operator was performing FNP-2-STP-33.1. While testing the feedwater isolation signal, he received an improper reset indication for the 2A steam generator feedwater isolation signal but continued with the test believing that the improper indication would clear when he went to NORMAL. When he placed the test switch in the NORMAL position, the 2A steam generator feed regulating valve closed causing the 2A steam generator level to decrease to the Lo Level set-point (25%). The reactor tripped at 1107 due to Lo Level in 2A steam generator in conjunction with Loop 1 Steam Flow Greater Than Feed Flow.

The plant operators implemented FNP-2-EOP-5 (Reactor Trip) placing the unit safely in Mode 3. All safety systems functioned as designed.

This event was caused by personnel error. The plant operator continued FNP-2-STP-33.1 even though he had not received a proper reset indication for the 2A steam generator feedwater isolation signal. The individual involved was counseled concerning the importance of adherence to procedures. Investigation revealed that a small bit of foreign material was obstructing the contacts on relay K620 preventing proper reset of the feedwater isolation signal. The relay was replaced.