

1983 ANNUAL ENVIRONMENTAL REPORT
RADIOLOGICAL - VOLUME #2

DUQUESNE LIGHT COMPANY
BEAVER VALLEY POWER STATION
AND
SHIPPINGPORT ATOMIC POWER STATION

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DUQUESNE LIGHT COMPANY
1983 Annual Radiological Environmental Report

ABSTRACT

This report describes the Radiological Environmental Monitoring Program conducted during 1983 in the vicinity of the Beaver Valley Power Station and the Shippingport Atomic Power Station. The Radiological Environmental Program consists of on-site sampling of water and gaseous effluents and off-site monitoring of water, air, river sediments, soils, food pathway samples, and radiation levels in the vicinity of the site. This report discusses the results of this monitoring during 1983.

The environmental program outlined in the Beaver Valley Power Station Technical Specifications was followed throughout 1983. The results of this environmental monitoring program show that Shippingport Atomic Power Station and Beaver Valley Power Station operations have not adversely affected the surrounding environment.

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SECTION I

DUQUESNE LIGHT COMPANY 1983 Annual Radiological Environmental Report

I. INTRODUCTION

The 1983 Annual Radiological Environmental Report for the Beaver Valley Power Station and the Shippingport Atomic Power Station summarizes the radiological environmental program conducted by the Duquesne Light Company in 1983.

The Duquesne Light Company operates the Shippingport Atomic Power Station for the United States Department of Energy and the Beaver Valley Power Station pressurized water reactor - Unit No. 1 as part of the Central Area Power Coordination group. Beaver Valley No. 2 Unit was under construction in 1983 and is scheduled to start-up in 1986.

The Shippingport Atomic Power Station was permanently shutdown on October 1, 1982. During 1983 the Shippingport Atomic Power Station nuclear reactor core was disassembled for shipment off-site for disposal.

The Shippingport Atomic Power Station was the first large-scale central station nuclear reactor in the United States. Since initial power generation in December 1957, operation of the pressurized water reactor at the Shippingport plant has supplied power to the Duquesne Light Company system in addition to providing technology which has served as a basis for the development of pressurized water reactors in the nuclear industry. The plant utilized a Light Water Breeder Reactor (LWBR) core from September 21, 1977 until final shutdown on October 1, 1982, pioneering light water breeder technology.

The highest average daily output generated at the Beaver Valley Power Station during the year was 817 megawatts net in April, 1983. The total gross electrical generation during the year was 4,916,300 megawatt-hours.

SECTION I

DUQUESNE LIGHT COMPANY 1983 Annual Radiological Environmental Report

I. INTRODUCTION

A. Scope and Objectives of the Program

The environmental program consists of effluent and environmental monitoring for radioactivity. Liquid and gaseous effluents from the Beaver Valley Power Station and the Shippingport Atomic Power Station were collected, processed, sampled, and analyzed to ensure conformance with the applicable regulations and permits prior to their release to the environment. Environmental sampling and analyses included air, water, milk, soil, vegetation, river sediments, fish, and ambient radiation levels in areas surrounding both plants.

B. Description of the Shippingport and Beaver Valley Site

The Shippingport Atomic Power Station and the Beaver Valley Power Station are located on the south bank of the Ohio River in the Borough of Shippingport, Beaver County, Pennsylvania, on a 486.8 acre tract of land which is owned by the Duquesne Light Company. Figure 1.0 is an artist's view of both stations. The site is approximately one mile from Midland, Pennsylvania; 5 miles from East Liverpool, Ohio; and 25 miles from Pittsburgh, Pennsylvania. Figure 1.1 shows the site location in relation to the principal population centers. Population density in the immediate vicinity of the site is relatively low. There are no residents within a 1/2 mile radius of either plant. The population within a 5 mile radius of the plant is approximately 18,000 and the only area within that radius of concentrated population is the Borough of Midland, Pennsylvania, with a population of approximately 4,300.

The site lies in a valley along the Ohio River. It extends from the river (elevation 665 feet above sea level) to a ridge along the border south of the Shippingport and Beaver Valley Power Stations at an elevation of 1,160 feet. Plant ground level at both stations is approximately 735 feet above sea level.

The two (2) stations are situated on the Ohio River at river mile 34.8, at a location on the New Cumberland Pool that is 3.3 river miles downstream from Montgomery Lock and Dam, and 19.4 miles upstream from New Cumberland Lock and Dam. The Pennsylvania-Ohio-West Virginia border is located 5.2 river miles downstream from the site. The river flow is regulated by a series of dams and reservoirs on the Beaver, Allegheny, Monongahela and Ohio Rivers and their tributaries. Flow ranges from a minimum of 5000 cubic feet per second (CFS) to a maximum of 100,000 CFS. The mean annual flow is approximately 25,000 CFS.

SECTION I

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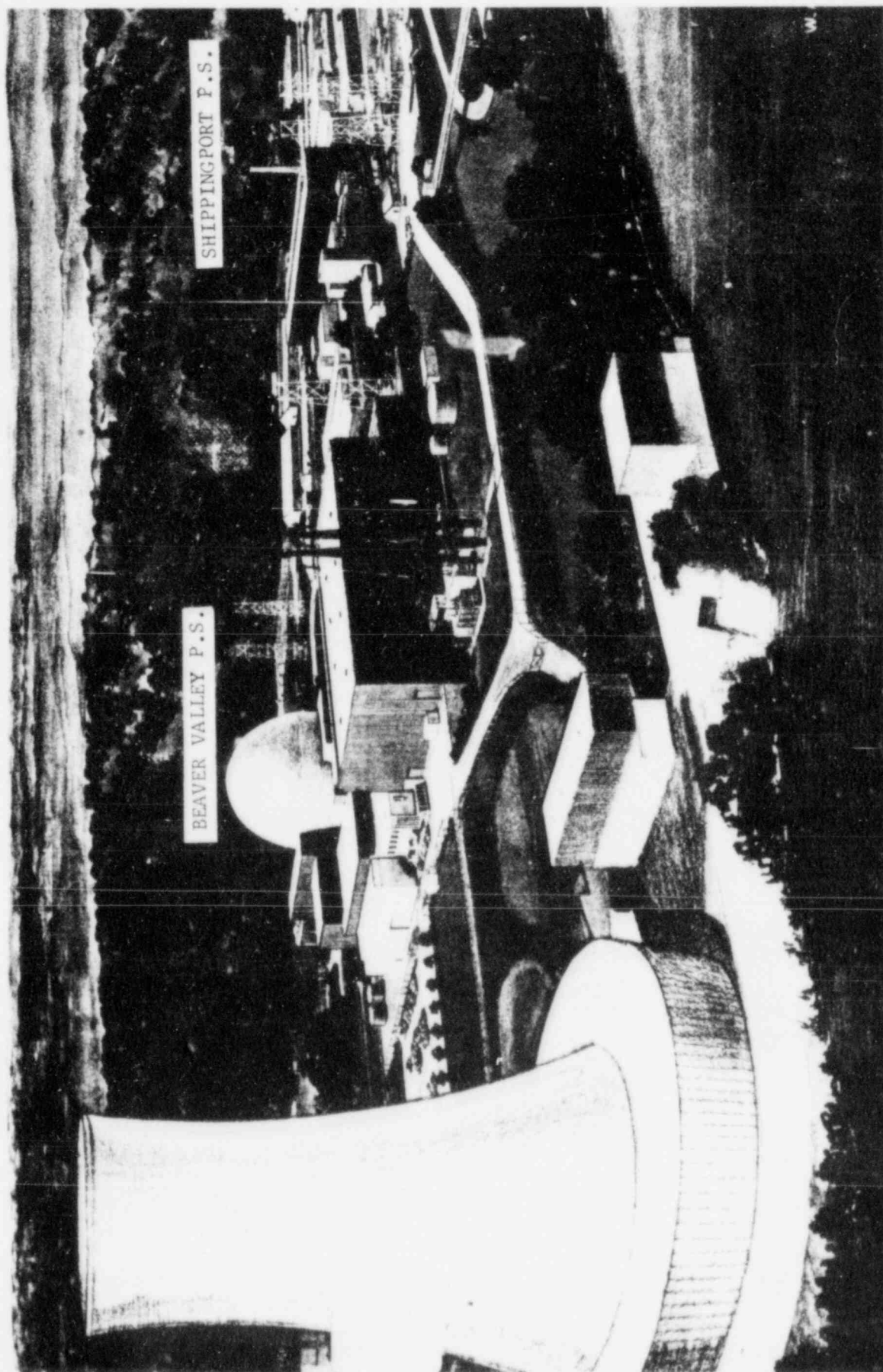
I. INTRODUCTION

B. Description of the Shippingport and Beaver Valley Site
(continued)

Water temperature of the Ohio River varies from 32°F to 84°F, the minimum temperatures occur in January and/or February and maximum temperatures in July and August. Water quality in the Ohio River at the site location is affected primarily by the water quality of the Allegheny, Monongahela, and Beaver rivers.

The climate of the area may be classified as humid continental. Annual precipitation is approximately 33 inches, typical yearly temperatures vary from approximately -3°F to 95°F with an annual average temperature of 52.8°F. The predominant wind direction is typically from the southwest in summer and from the northwest in winter.

FIGURE 1.0



VIEW OF THE BEAVER VALLEY POWER STATION AND THE SHIPPINGPORT ATOMIC POWER STATION

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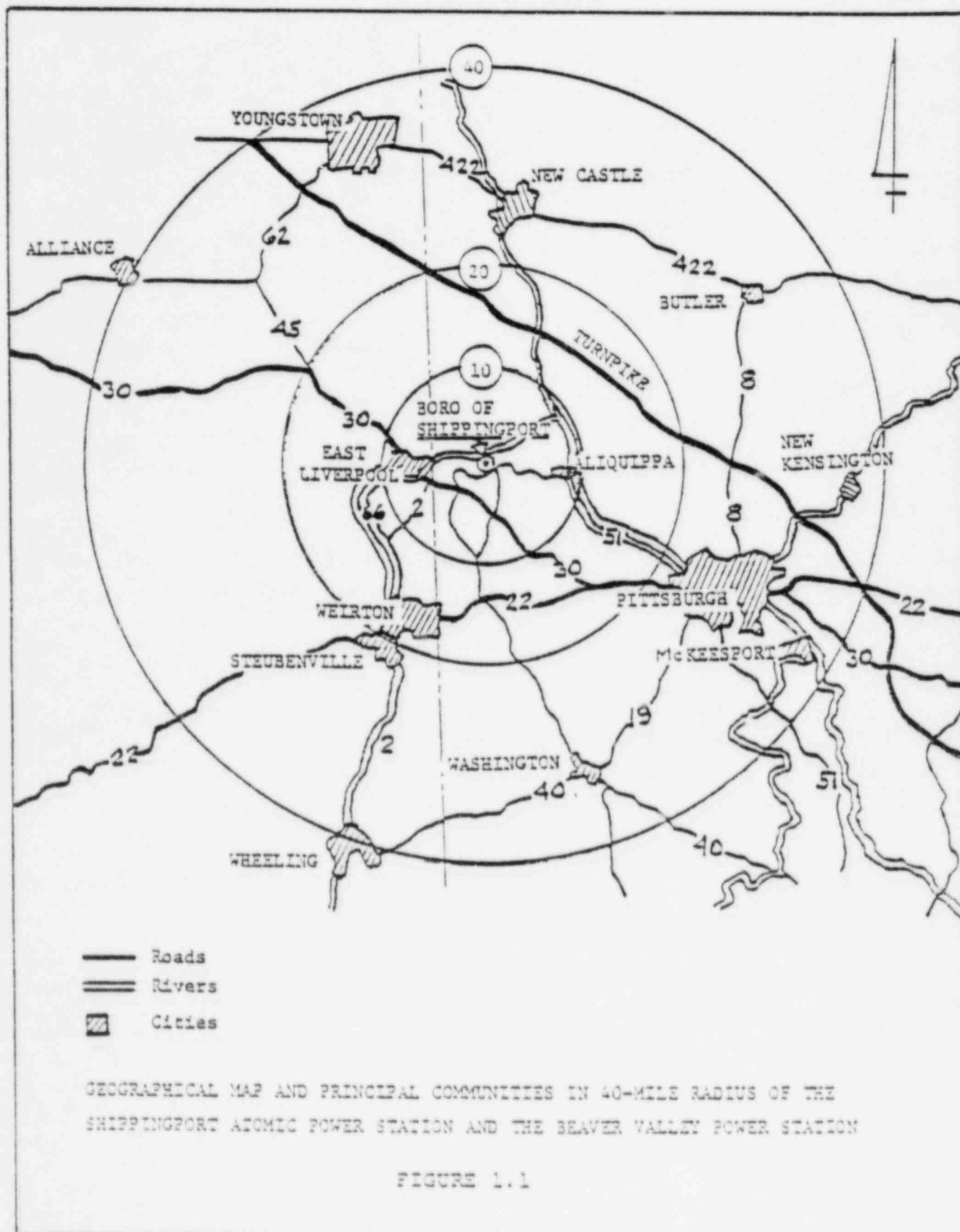


FIGURE 1.1

SECTION I

DUQUESNE LIGHT COMPANY
1983 Annual Radiological Environmental ReportI. INTRODUCTIONB. Description of the Shippingport and Beaver Valley Site
(continued)

The design ratings and basic features of the Beaver Valley Power Station and the Shippingport Atomic Power Station are tabulated below:

	<u>Beaver Valley</u>	<u>Shippingport</u> ⁽¹⁾
Thermal & Elec. Rating - 2660 MW _t 835 MW _e MW-Each Reactor		236.6 MW _t 72 MW _e
Type of Reactor	PWR	PWR ⁽²⁾
Number of Reactor Coolant Loops	3	4
Number of Steam Generators and Type	3 - Vertical	4 - Horizontal
Steam Used by Main Turbine	Saturated	Saturated

Both stations utilize two (2) separate systems (primary and secondary) for transferring heat from the source (the reactor) to the receiving component (turbine-generator). Because the two systems are isolated from each other, primary and secondary waters do not mix; therefore, radioactivity in the primary system water is normally isolated from the secondary system. Reactor coolant in the primary system is pumped through the reactor core and steam generators by means of reactor coolant pumps. Heat is given up from the primary system to the secondary system in the steam generators, where steam is formed and delivered to the main unit turbine, which drives the electrical generator. The steam is condensed after passing through the turbine, and returned to the steam generators to begin another steam/water cycle.

NOTE: MW_t - megawatts thermal
MW_e - megawatts electrical

(1) The Shippingport Atomic Power Station was not operated during 1983.

(2) Light Water Breeder Core

II. RESULTS AND CONCLUSIONS

Plant operations at both the Beaver Valley Power Station and the Shippingport Atomic Power Station had no adverse effects on the environment as a result of activities at either of the stations during 1983. Comparisons of pre-operational data with operational data indicates the ranges of values are in good agreement for both periods of time.

The Beaver Valley Power Station operated throughout 1983, while the Shippingport Atomic Power Station was shutdown and defueled throughout 1983. During the year, the radioactive releases from both stations were below the limits of 10 CFR Part 50, Appendix I and applicable permits for each station. The releases at Beaver Valley Power Station did not exceed the limiting conditions identified in the Beaver Valley Power Station Operating License Technical Specifications.

The environmental program for 1983 was the same as in 1982 except for several changes in dairy locations which were revised as required by the Beaver Valley Technical Specifications. (Refer to Table V.A.1 for the 1983 Radiological Monitoring Program Outline).

The results of the 1983 Radiological Environmental Monitoring Program are consistent with those of previous years. The only radioactivity above normal ambient levels in the environs other than world-wide fallout from Nuclear weapons tests was detected near the Beaver Valley Power Station discharge and resulted in negligible exposure to members of the public. A summary of the 1983 operational environmental data (ranges and means) for each sampling media is found in Table V.A.2. A summary of preoperational (1974 - 1975) environmental data is found in Table V.A.3.

The Beaver Valley Power Station Technical Specifications require sampling of three (3) dairies which have the highest calculated milk pathway potential and one large local dairy. The three dairies are determined from calculations based on the meteorological data and the latest milch animal survey. However, these dairies are frequently small, consisting of as few as one cow or goat. The availability of milk from single cow dairies and revisions due to updated calculations and surveys result in sampling of several additional dairies during the year in different sampling periods.

SECTION II

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II. RESULTS AND CONCLUSIONS (continued)

In addition, the Environmental Monitoring Program includes two larger dairies in order to provide continuity in the sampling/analyses program and a control location. Samples from each of these dairies are obtained each month in addition to the four dairies required by the Environmental Technical Specifications. During some sampling periods, one or all of the additional dairies could be among the required dairies. The collection periods associated with each of the locations are provided in the detailed summary of the milk monitoring program of this report (Section V-E).

Examination of effluents from the Shippingport Atomic Power Station and the Beaver Valley Power Station and environmental media demonstrated compliance with regulations and Station Technical Specifications.

III. ENVIRONMENTAL MONITORING CONSIDERATIONSA. Environmental Quality Control Programs

The Quality Control (QC) Program used for the Beaver Valley - Shippingport Environmental Radioactivity Monitoring Program consisted of seven (7) elements. It should be noted that the comparisons made were at very low levels of radioactivity and consequently, the activities at these levels are difficult to measure. However, acceptable correlation was achieved in most instances as outlined in the discussions and tables which follow.

1. Radiation Monitoring (Duquesne Light Company (DLC) Contractor Laboratory - DLC QC Laboratory - Independent Laboratory)

An independent program of external radiation monitoring was conducted by the QC Laboratory using lithium fluoride TLDs sharing the same location as the DLC Contractor Laboratory TLDs and Independent Laboratory TLDs. Summary data of the QC Laboratory program is provided in Table III.1.

Duplicate contractor TLD, QC TLD, Annual TLD, and Independent Lab TLD and continuous integrating monitoring by a Pressurized Ion Chamber (PIC) show generally good agreement and demonstrate acceptable performance by the DLC Contractor Laboratory. The arithmetic mean of each laboratory agrees within $\pm 5\%$ of the arithmetic mean of the three laboratories. This is well within the precision of a typical TLD system.

2. Split Sample Program (DLC Contractor Laboratory - DLC QC Laboratory)

Samples of surface (river) water and drinking water were routinely split and analyzed by the DLC Contractor Laboratory and the DLC QC Laboratory. In addition, samples of other media, such as milk, soil, sediment and feedcrop were also split with the DLC QC Laboratory (a laboratory of the Department of Energy).

A summary of results of split water samples is provided in Table III.2. A summary of milk, sediment and feed/food crop split samples is provided in Table III.3. Some variation is expected due to small variations in duplicate samples, variations in analytical procedures, and in calibration, source type, etc.

SECTION III

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III. ENVIRONMENTAL MONITORING CONSIDERATIONS

A. Environmental Quality Control Programs (continued)

2. Split Sample Program (DLC Contractor Laboratory - DLC QC Laboratory (continued))

Because of the overall uniformity of comparable results, it is concluded that the two laboratories are consistent and in agreement.

TABLE III.1
QUALITY CONTROL RESULTS
TLD MONITORING
mR/Day

1ST QUARTER						2ND QUARTER				
Location No.	DLC Contractor (CaSO ₄ :Dy)	DLC QC Lab (LiF)	Independent Lab #3 (CaSO ₄ :Tm)	PDER (2)	DLC PIC (3)	DLC Contractor (CaSO ₄ :Dy)	DLC QC Lab (LiF)	Independent Lab #3 (CaSO ₄ :Tm)	PDER (2)	DLC PIC (3)
10	0.18	0.22	0.21	0.26	0.21 ⁽¹⁾	0.17	0.19	0.19	0.27	0.21 ⁽¹⁾
13	0.16	0.22	0.22	0.27		0.16	0.18	0.1	0.29	
14	0.17	0.22	0.20			*	*	0.19		
15	0.14	0.16	0.18	0.22		0.13	0.13	0.16	0.27	
27	0.18	0.21	0.20			0.16	0.16	0.18		
28	0.19	0.23	0.20			0.17	0.20	0.20		
29	0.20	0.28	0.25			0.20	0.23	0.22		
32	0.20	0.27	0.24	0.23	0.24	0.18	0.20	0.22	0.23	0.24
45	0.18	0.24	0.23	0.29	0.26 ⁽¹⁾	0.19	0.21	0.20	0.29	0.25 ⁽¹⁾
46	0.16	0.20	0.20	0.25	0.19 ⁽¹⁾	0.13	0.13	0.17	0.27	0.19 ⁽¹⁾
47	0.19	0.24	0.23			0.19	0.21	0.21		
48	0.20	0.24	0.20			0.18	0.18	0.20		
51	0.19	0.21	0.24	0.25		0.17	0.19	0.18	0.30	
3RD QUARTER						4TH QUARTER				
Location No.	DLC Contractor (CaSO ₄ :Dy)	DLC QC Lab (LiF)	DLC Contractor*** Independent Lab #3 (CaSO ₄ :Tm)	PDER (2)	DLC PIC (3)	DLC Contractor (CaSO ₄ :Dy)	DLC QC Lab (LiF)	DLC Contractor*** Independent Lab #3 (CaSO ₄ :Tm)	PDER (2)	DLC PIC (3)
10	0.18	0.16	0.16	0.25	0.26 ⁽¹⁾	0.16	0.20	0.19	0.22	0.25 ⁽¹⁾
13	0.17	0.17	0.16	0.25		0.16	0.17	0.17	0.25	
14	0.18	0.18	0.18			0.16	0.20	0.21		
15	0.14	0.12	0.13	0.21		0.13	0.14	0.14	0.21	
27	0.18	0.16	0.17			0.16	0.18	0.18		
28	0.19	0.19	0.18			0.17	0.21	0.20		
29	0.21	0.22	0.20			0.18	0.23	0.21		
32	0.20	0.18	0.19	0.23	**	0.16	0.21	0.20	0.19	**
45	0.19	0.18	0.17	0.26	0.25 ⁽¹⁾	0.17	0.19	0.20	0.25	0.25 ⁽¹⁾
46	0.13	0.15	0.17	0.22	0.19 ⁽¹⁾	0.15	0.17	0.14	0.22	0.19 ⁽¹⁾
47	0.21	0.19	0.20			0.20	0.20	0.21		
48	0.18	0.16	0.17			0.20	0.19	0.20		
51	0.21	0.19	0.18	0.26		0.19	0.20	0.19	0.27	

(1) PIC Reading at Location 10 taken in DLC Substation in Shippingport Boro, Location 45 taken at Kennedy's Corners, Location 46 taken at Industry Tire Shop.

(2) NRC readings from Pennsylvania Department of Environmental Resources.

(3) In this consolidated environmental program the pressurized ion chamber (PIC) continuous monitor readings tend to be slightly higher than the TLD readings due to the differences in the inherent physics of each system. No compensatory measures have been taken to make both systems agree exactly because both systems were installed to monitor relative radiation levels rather than absolute levels. Each system provides a reasonably accurate measure of the absolute radiation levels.

* TLD missing from its posted location

** Station is out of service

*** 3rd and 4th quarter Independent Laboratory equipment taken over by Contractor lab.

TABLE III.2
QUALITY CONTROL RESULTS
SPLIT SAMPLE ANALYSIS RESULTSComparison of Contractor and DLC-QC Labs

<u>Media</u>	<u>Analysis</u>	<u>Sampling Period</u>	<u>DLC Contractor Lab (1)</u>	<u>DLC - QC Lab (1)</u>	<u>Units</u>
Surface Water	Gross Alpha	January	2.6 ± 1.3	2.9 ± 2.5	pCi/l
		April	≤ 0.71	≤ 1.0	pCi/l
		July	≤ 1.1	≤ 1.2	pCi/l
		October	≤ 1.4	≤ 1.4	pCi/l
Surface Water	Gross Beta	January	11 ± 2	9.0 ± 2.2	pCi/l
		April	7.7 ± 1.5	4.8 ± 1.7	pCi/l
		July	5.0 ± 1.2	5.0 ± 1.7	pCi/l
		October	6.6 ± 1.4	5.9 ± 2.5	pCi/l
Surface Water	Co-60	January	≤ 2.0	≤ 4.2	pCi/l
		April	≤ 2.0	≤ 3.7	pCi/l
		July	≤ 3.0	≤ 3.9	pCi/l
		October	≤ 1.0	≤ 3.7	pCi/l
Surface Water	Cs-134	January	≤ 2.0	≤ 5.0	pCi/l
		April	≤ 3.0	≤ 4.4	pCi/l
		July	≤ 2.0	≤ 4.5	pCi/l
		October	≤ 1.0	≤ 4.2	pCi/l
Surface Water	Cs-137	January	≤ 3.0	≤ 5.0	pCi/l
		April	≤ 3.0	≤ 4.7	pCi/l
		July	≤ 3.0	≤ 4.7	pCi/l
		October	≤ 2.0	≤ 3.7	pCi/l
Surface Water	Tritium	1st Quarter Composite	1030 ± 100	900 ± 320	pCi/l
		3rd Quarter Composite	2290 ± 140	2610 ± 280	pCi/l
Surface Water	Sr-89	2nd Quarter Composite	≤ 1.7	≤ 0.36	pCi/l
		4th Quarter Composite	≤ 1.2	≤ 3.0	pCi/l
Surface Water	Sr-90	2nd Quarter Composite	≤ 0.29	≤ 0.36	pCi/l
		4th Quarter Composite	≤ 0.32	≤ 0.48	pCi/l
Surface Water	Co-60 (high sensitivity analysis)	2nd Quarter Composite	≤ 1.0	≤ 3.6	pCi/l
		4th Quarter Composite	≤ 0.5	≤ 4.9	pCi/l

(1) Uncertainties are based on counting statistics and are specified at the 95% confidence level.

TABLE III.2 (Continued)
QUALITY CONTROL RESULTS
SPLIT SAMPLE ANALYSIS RESULTSComparison of Contractor and DLC-QC Labs

<u>Media</u>	<u>Analysis</u>	<u>Sampling Period</u>	<u>DLC Contractor Lab (1)</u>	<u>DLC - QC Lab (1)</u>	<u>Units</u>
Drinking Water	Cs-137	February	≤ 2.0	≤ 5.0	pCi/l
		May	≤ 3.0	≤ 5.0	pCi/l
		August	≤ 2.0	≤ 5.4	pCi/l
		November	≤ 2.0	≤ 5.0	pCi/l
Drinking Water	Cs-134	February	≤ 2.0	≤ 5.0	pCi/l
		May	≤ 3.0	≤ 5.0	pCi/l
		August	≤ 2.0	≤ 5.3	pCi/l
		November	≤ 2.0	≤ 5.0	pCi/l
Drinking Water	Co-60	February	≤ 2.0	≤ 4.8	pCi/l
		May	≤ 3.0	≤ 4.0	pCi/l
		August	≤ 2.0	≤ 4.6	pCi/l
		November	≤ 2.0	≤ 4.4	pCi/l
Drinking Water	Gross Alpha	March	≤ 0.39	≤ 1.0	pCi/l
		June	≤ 0.45	≤ 1.6	pCi/l
		August	≤ 0.86	≤ 1.6	pCi/l
		November	≤ 0.46	≤ 1.9	pCi/l
Drinking Water	Gross Beta	March	4.1 ± 1.2	6.1 ± 2.0	pCi/l
		June	3.0 ± 0.9	2.6 ± 1.7	pCi/l
		August	4.1 ± 1.4	6.2 ± 3.7	pCi/l
		November	3.7 ± 1.4	2.7 ± 1.8	pCi/l
Drinking Water	Tritium	2nd Quarter	270 ± 80	≤ 200	pCi/l
		4th Quarter	≤ 70	≤ 170	pCi/l

(1) Uncertainties are based on counting statistics and are specified at the 95% confidence interval.

TABLE III.3
QUALITY CONTROL RESULTS
SPLIT SAMPLE ANALYSIS RESULTSComparison of Contractor and DLC-QC Labs

Media	Analysis	Sampling Period	DLC Contractor Lab (1)	DLC - QC Lab (1)	Units
Milk (Location 25)	I-131	3-29-83	≤ 0.13	≤ 0.1	pCi/l
	I-131	6-21-83	≤ 0.03	≤ 0.23	pCi/l
	K-40	6-21-83	1260 ± 130	--	pCi/l
	Cs-134	6-21-83	≤ 1.0	≤ 3.0	pCi/l
	Cs-137	6-21-83	≤ 3.0	≤ 3.0	pCi/l
	Co-60	6-21-83	≤ 3.0	≤ 4.3	pCi/l
Feed (Location 25)	K-40	6-21-83	12.3 ± 1.2	--	pCi/gm Dry
	Co-60	6-21-83	≤ 0.01	≤ 0.048	pCi/gm Dry
	Cs-134	6-21-83	≤ 0.01	≤ 0.050	pCi/gm Dry
	Cs-137	6-21-83	≤ 0.02	≤ 0.072	pCi/gm Dry
	Th-232	6-21-83	0.09 ± 0.03	--	pCi/gm Dry
Feed (Location 25)	Sr-90	6-21-83	0.043 ± 0.009	0.24 ± 1.12	pCi/gm Dry
Food (Cabbage) (Location 10)	I-131	9-1-83	≤ 0.003	--	pCi/gm Wet
	K-40	9-1-83	1.88 ± 0.19	--	pCi/gm Wet
	Co-60	9-1-83	≤ 0.004	≤ 0.12	pCi/gm Wet
	Cs-134	9-1-83	≤ 0.003	≤ 0.12	pCi/gm Wet
	Cs-137	9-1-83	≤ 0.003	≤ 0.13	pCi/gm Wet
Food (Swiss chard) (Location 10)	I-131	9-1-83	0.163 ± 0.066	--	pCi/gm Wet
	K-40	9-1-83	3.69 ± 0.57	--	pCi/gm Wet
	Co-60	9-1-83	≤ 0.003	≤ 0.11	pCi/gm Wet
	Cs-134	9-1-83	≤ 0.004	≤ 0.13	pCi/gm Wet
	Cs-137	9-1-83	0.02 ± 0.007	≤ 0.12	pCi/gm Wet

(1) Uncertainties are based on counting statistics and are specified at the 95% confidence level.

-- Analyses not performed nor required.

TABLE III.3 (Continued)
QUALITY CONTROL RESULTS
SPLIT SAMPLE ANALYSIS RESULTSComparison of Contractor and DLC-QC Labs

<u>Media</u>	<u>Analysis</u>	<u>Sampling Period</u>	<u>DLC Contractor Lab (1)</u>	<u>DLC - QC Lab (1)</u>	<u>Units</u>
Sediment (Location 3)	Gross Alpha	10-25-83	14 \pm 7	12 \pm 6.5	pCi/gm Dry
	Gross Beta	10-25-83	29 \pm 3	29 \pm 7.8	pCi/gm Dry
	U-235	10-25-83	0.015 \pm 0.004	0.23 \pm 0.06	pCi/gm Dry
	U-234	10-25-83	0.83 \pm 0.08	0.80 \pm 0.13	pCi/gm Dry
	U-238	10-25-83	0.62 \pm 0.06	0.53 \pm 0.10	pCi/gm Dry
	Sr-89	10-25-83	\leq 0.14	\leq 0.22	pCi/gm Dry
	Sr-90	10-25-83	0.041 \pm 0.025	\leq 0.11	pCi/gm Dry
	K-40	10-25-83	15.7 \pm 1.6	--	pCi/gm Dry
	Co-60	10-25-83	0.239 \pm 0.07	0.30 \pm 0.21	pCi/gm Dry
	Cs-134	10-25-83	\leq 0.03	\leq 0.16	pCi/gm Dry
	Cs-137	10-25-83	0.497 \pm 0.065	0.42 \pm 0.097	pCi/gm Dry
	Ra-226	10-25-83	2.78 \pm 1.01	--	pCi/gm Dry
	Th-228	10-25-83	1.56 \pm 0.16	--	pCi/gm Dry
Milk (Location 25)	Sr-89	12-11-83	\leq 1.0	\leq 0.92	pCi/l
	Sr-90	12-11-83	1.4 \pm 0.5	2.4 \pm 0.78	pCi/l
	Co-60	12-11-83	\leq 2.0	\leq 4.4	pCi/l
	Cs-134	12-11-83	\leq 3.0	\leq 5.0	pCi/l
	Cs-137	12-11-83	\leq 2.0	\leq 5.0	pCi/l
	K-40	12-11-83	1270 \pm 130	--	pCi/l
	I-131	12-11-83	\leq 0.24	\leq 0.28	pCi/l

(1) Uncertainties are based on counting statistics and are specified at the 95% confidence level.

-- Analyses not performed nor required.

A. Environmental Quality Control Programs (continued)3. DLC QC Laboratory Program

Spiked samples prepared by DLC QC Laboratory were routinely submitted to the Contractor Laboratory for analysis. Tables III.4 (water) and III.5 (milk) provide data from this portion of the QC Program. The overall results demonstrate that the contractor performed acceptably in the program.

4. Comparisons of Similar Samples (DLC Contractor Laboratory - DLC QC Laboratory)

Duplicate air particulate and charcoal filters (radioiodine) samples were collected at Location #30 and compared during the year on a weekly basis. Comparison of particulate and charcoal samples alternated from week to week. Duplicate monthly air particulate filters, composited from the weekly air particulate filters, were analyzed 6 months out of the year for gamma activity. Duplicate quarterly air particulate filters, composited from the weekly air particulate filters, were analyzed for Sr-89 and Sr-90 activity for the first and third quarters of the year. Table III.6 provides data for this portion of the QC program. The results show generally good agreement between the laboratories and demonstrate that the contractor performed acceptably in the program.

5. Contractor Internal QC Program

The Contractor Laboratory maintained its own QC Program which included participation in the Environmental Protection Agency - Environmental Monitoring Safety Laboratory (EPA - EMSL) Interlaboratory Cross Check Program. This cross check program indicated that the Contractor results were in agreement with EPS EMSL. DLC also audited the Contractor Laboratory and determined that internal QC practices were in effect and that procedures and laboratory analytical techniques conformed to approved DLC procedures.

A. Environmental Quality Control Programs (continued)6. Special QC Program (DLC Contractor Laboratory -
Independent Laboratory - DLC QC Laboratory)

Milk and water samples were prepared quarterly by an Independent Laboratory. This included low level spiking of specified nuclides. The prepared samples were split three ways and analyzed by the DLC-QC Laboratory and Independent Laboratory as well as the Contractor Laboratory. A summary of results of this portion of the QC program is provided in Table III.7. The results show generally good agreement between the laboratories and demonstrate that the contractor performed acceptably in the program.

TABLE III.4
QUALITY CONTROL RESULTS
SPIKE SAMPLE ANALYSIS RESULTS

Sample Date	Ident. No.	Sample Type and Analysis	DLC Contractor Lab (1)	DLC - QC Lab (1)	Units
4-1-83	53-52	Water: Sr-89	6.6 ± 1.6	3.8 ± 1.3	pCi/l
		Sr-90	≤ 0.4	≤ 0.4	pCi/l
4-1-83	53-52	Water: I-131	13 ± 2	12 ± 6	pCi/l
		Cr-51	25.3 ± 5.2	27.0 ± 6.0	pCi/l
		Ca-134	26 ± 6.9	19 ± 7	pCi/l
		Ce-144	83.2 ± 41.3	66 ± 46	pCi/l
7-29-83	53-54	Water: Gross Alpha	8.6 ± 1.4	12 ± 1.7	pCi/l
		Gross Beta	$25 \pm 2^{**}$	41 ± 2.8	pCi/l
9-1-83	53-55	Water: Sr-89	$\leq \text{LLD}^*$	11 ± 2.4	pCi/l
		Sr-90	$6.3 \pm 0.9^*$	13 ± 1.6	pCi/l
		Co-58	39.7 ± 8.7	36 ± 9	pCi/l
		Co-60	14.0 ± 3.4	16 ± 6.6	pCi/l
10-20-83	53-56	Water: I-131	20 ± 1	26 ± 2.8	pCi/l
		Co-58	119 ± 12	99 ± 9.6	pCi/l
		Co-60	37.1 ± 5.9	58 ± 8.8	pCi/l
		Ce-144	232 ± 29	170 ± 64	pCi/l
		H-1	2050 ± 130	2000 ± 280	pCi/l
1-24-84	53-57	Water: Gross Alpha	$2.5 \pm 0.9^{**}$	13 ± 3.7	pCi/l
		Gross Beta	32 ± 2	40 ± 1.9	pCi/l
2-15-84	53-A	Water: Sr-89	7.9 ± 1.7	11 ± 1.8	pCi/l
		Sr-90	11 ± 1	8.2 ± 1.4	pCi/l
	53-B	Water: Sr-89	21 ± 3	22 ± 2.8	pCi/l
		Sr-90	23 ± 1	20 ± 2.1	pCi/l
	53-C	Water: Sr-89	31 ± 2	30 ± 3.3	pCi/l
		Sr-90	30 ± 2	30 ± 2.4	pCi/l

(1) Uncertainties are based on counting statistics and are specified at the 95% confidence level.

LLD Lower limit of detector.

* Four of five spiked samples are in good agreement.

** While these samples are not in good agreement with the Quality Control Laboratory, the Contractor Laboratory did agree with nine EPA Intercomparison Laboratory results ranging from 10 pCi/l to 130 pCi/l.

TABLE III.5
QUALITY CONTROL RESULTS
SPIKE SAMPLE ANALYSIS

<u>Sample Date</u>	<u>Ident. No.</u>	<u>Sample Type and Analysis</u>	<u>DLC Contractor Lab (1)</u>	<u>DLC - QC Lab (1)</u>	<u>Units</u>
1-17-83	52-57	Milk: Sr-89	≤ 3.0	7.4 ± 5.2	pCi/l
		Sr-90	6.6 ± 0.9	6.3 ± 3.5	pCi/l
		Mn-54	42.1 ± 6.5	41 ± 7	pCi/l
		Cs-134	18.2 ± 7	11 ± 6	pCi/l
		Cs-137	29.1 ± 6	31 ± 9	pCi/l
		I-131	4.5 ± 0.3	6.8 ± 1.2	pCi/l
6-8-83	52-58	Milk: I-131	10 ± 1	7.6 ± 1.0	pCi/l
		Fe-59	31 ± 10.8	29 ± 18	pCi/l
		Cs-134	53 ± 5.3	48 ± 11	pCi/l
10-20-83	52-60	Milk: I-131	8.3 ± 1.5	12 ± 1.4	pCi/l
		Mn-54	40.5 ± 4.4	39 ± 11	pCi/l
		Cs-60	33.6 ± 5.1	24 ± 8.6	pCi/l
		Cs-137	72 ± 7.2	59 ± 9	pCi/l

(1) Uncertainties are based on counting statistics and are specified at the 95% confidence level.

TABLE III.6
QUALITY CONTROL RESULTS
AIR PARTICULATES AND CHARCOAL FILTER: COMPARABLE SAMPLES

Air Particulates pCi/Cu. Meter (Beta)			Air Iodine pCi/Cu. Meter		
Sample Date	DLC Contractor Lab (1)	DLC - QC Lab (1)	Sample Date	DLC Contractor Lab (1)	DLC - QC Lab (1)
1-3-83 to 1-10-83	0.022 ± 0.003	0.030 ± 0.003	1/10/83 to 1-17-83	≤ 0.01	≤ 0.027
1-17-83 to 1-24-83	0.013 ± 0.003	0.018 ± 0.002	1-24-83 to 1-31-83	≤ 0.009	≤ 0.027
1-31-83 to 2-7-83	0.015 ± 0.003	0.023 ± 0.003	2-7-83 to 2-14-83	≤ 0.01	≤ 0.017
2-14-83 to 2-21-83	0.022 ± 0.004	0.037 ± 0.003	2-22-83 to 2-28-83	≤ 0.01	≤ 0.017
2-28-83 to 3-7-83	0.023 ± 0.003	0.029 ± 0.003	3-7-83 to 3-14-83	≤ 0.01	≤ 0.021
3-14-83 to 3-21-83	0.009 ± 0.003	0.014 ± 0.002	3-21-83 to 3-28-83	≤ 0.01	≤ 0.018
3-28-83 to 4-4-83	0.012 ± 0.004	0.022 ± 0.002	4-4-83 to 4-11-83	≤ 0.009	≤ 0.013
4-11-83 to 4-18-83	0.011 ± 0.003	0.015 ± 0.003	4-18-83 to 4-25-83	≤ 0.009	≤ 0.012
4-25-83 to 5-2-83	0.017 ± 0.003	0.023 ± 0.002	5-2-83 to 5-9-83	≤ 0.01	≤ 0.010
5-9-83 to 5-16-83	0.011 ± 0.003	0.019 ± 0.002	5-16-83 to 5-23-83	≤ 0.005	≤ 0.019
5-23-83 to 5-31-83	0.010 ± 0.003	0.015 ± 0.002	5-31-83 to 6-6-83	≤ 0.01	≤ 0.014

(1) Uncertainties are based on counting statistics and are specified at the 95% confidence level.

TABLE III.6 (Continued)
QUALITY CONTROL RESULTS
AIR PARTICULATES AND CHARCOAL FILTER: COMPARABLE SAMPLES

Air Particulates pCi/Cu. Meter (Beta)			Air Iodine pCi/Cu. Meter		
Sample Date	DLC Contractor Lab (1)	DLC + QC Lab (1)	Sample Date	DLC Contractor Lab (1)	DLC + QC Lab (1)
6-6-83 to 6-13-83	0.023 ± 0.004	0.030 ± 0.002	6-13-83 to 6-21-83	≤ 0.01	≤ 0.013
6-21-83 to 6-27-83	0.015 ± 0.004	0.025 ± 0.003	6-27-83 to 7-5-83	≤ 0.009	≤ 0.015
7-3-83 to 7-11-83	0.014 ± 0.004	0.018 ± 0.002	7-11-83 to 7-18-83	≤ 0.009	≤ 0.013
7-18-83 to 7-23-83	0.017 ± 0.003	0.024 ± 0.002	7-23-83 to 8-1-83	≤ 0.01	≤ 0.015
8-1-83 to 8-8-83	0.023 ± 0.003	0.032 ± 0.003	8-8-83 to 8-15-83	≤ 0.01	≤ 0.022
8-15-83 to 8-22-83	0.033 ± 0.004	0.042 ± 0.003	8-22-83 to 8-29-83	≤ 0.01	≤ 0.016
8-29-83 to 9-6-83	0.026 ± 0.003	0.036 ± 0.002	8-29-83 to 9-12-83	≤ 0.01	≤ 0.018
9-12-83 to 9-19-83	0.018 ± 0.003	0.029 ± 0.002	9-19-83 to 9-26-83	≤ 0.01	≤ 0.010
9-26-83 to 10-3-83	0.034 ± 0.004	0.042 ± 0.003	10-3-83 to 10-10-83	≤ 0.01	≤ 0.012
10-10-83 to 10-17-83	0.019 ± 0.003	0.022 ± 0.002	10-17-83 to 10-23-83	≤ 0.01	≤ 0.016
10-24-83 to 10-31-83	0.013 ± 0.003	0.019 ± 0.002	10-31-83 to 11-7-83	≤ 0.008	≤ 0.020
11-7-83 to 11-14-83	0.023 ± 0.003	0.027 ± 0.003	11-14-83 to 11-21-83	≤ 0.008	≤ 0.020
11-21-83 to 11-28-83	0.027 ± 0.004	0.037 ± 0.003	11-28-83 to 12-5-83	≤ 0.008	≤ 0.018
12-3-83 to 12-13-83	0.027 ± 0.003	0.024 ± 0.002	12-13-83 to 12-19-83	≤ 0.007	≤ 0.028
12-19-83 to 12-27-83	0.020 ± 0.003	0.021 ± 0.002	12-27-83 to 1-3-84	≤ 0.009	≤ 0.011

(1) Uncertainties are based on counting statistics and are specified at the 95% confidence interval.

TABLE III.6
QUALITY CONTROL
AIR PARTICULATES (pCi/m³)

<u>Sample Date</u>	<u>Nuclide</u>	<u>DLC Contractor Lab (1)</u>	<u>DLC - QC Lab (1)</u>
January (1/3/83 to 1/31/83)	Be-7	0.057 ± 0.018	0.051 ± 0.011
	Others	LLD	LLD
March (2/28/83 to 3/38/83)	Be-7	0.082 ± 0.027	0.066 ± 0.011
	Others	LLD	LLD
May (5/2/83 to 5/31/83)	Be-7	0.102 ± 0.021	0.075 ± 0.014
	Others	LLD	LLD
July (6/27/83 to 8/1/83)	Be-7	0.145 ± 0.020	0.100 ± 0.002
	K-40	0.027 ± 0.013	LLD
	Others	LLD	LLD
September (8/29/83 to 9/26/83)	Be-7	0.118 ± 0.027	0.086 ± 0.015
	Others	LLD	LLD
November (10/31/83 to 11/28/83)	Be-7	0.094 ± 0.022	0.063 ± 0.008
	Others	LLD	LLD

(1) Uncertainties are based on counting statistics and are specified at the 95% confidence interval.

LLD Lower limit of detector.

TABLE III.6
QUALITY CONTROLAIR PARTICULATE AND CHARCOAL FILTER - COMPARABLE SAMPLES
LOCATION 30 - ($\mu\text{Ci}/\text{m}^3$)

<u>Sample Date</u>	<u>Nuclide</u>	<u>DLC Contractor Lab (1)</u>	<u>DLC - QC Lab (1)</u>
1st Quarter Composite	Sr-89	≤ 0.0009	≤ 0.00029
12/27/83 to 3/28/83	Sr-90	≤ 0.0003	≤ 0.00014
3rd Quarter Composite	Sr-89	≤ 0.0004	≤ 0.0015
6/27/83 to 9/26/83	Sr-90	≤ 0.0001	≤ 0.00015

(1) Uncertainties are based on counting statistics and are specified at the 95% confidence interval.

TABLE III.7
QUALITY CONTROL DATAQC Sample Comparisons
(All Analyses in pCi/l)

<u>Sample Date</u>	<u>Ident. No.</u>	<u>Sample Type and Analyses</u>	<u>Independent Lab (1)</u>	<u>DLC Contractor Lab (1)</u>	<u>DLC - QC Lab (1)</u>
2-16-83	53-247	Water: Sr-90	14.6 ± 0.6	15 ± 1	15 ± 2
		I-131	16.1 ± 0.3	13 ± 1	13 ± 2
		Co-60	18 ± 8	21.9 ± 6.7	18 ± 6
		Cs-134	22 ± 8	19.4 ± 6.9	16 ± 6
		Cs-137	25 ± 8	18.5 ± 5.8	24 ± 6
2-16-83	53-248	Water: H-3	1170 ± 60	1160 ± 100	1320 ± 330
5-25-83	53-249	Water: Sr-90	22.4 ± 0.6	20 ± 1	21 ± 1.8
		I-131	23.3 ± 1.4	21 ± 1	13 ± 1.3
		Mn-54	38 ± 5	29.8 ± 4	29 ± 8.6
		Cs-134	15 ± 4	16.5 ± 4.1	19 ± 11
		Cs-137	16 ± 3	19.6 ± 4.4	19 ± 8.2
5-25-83	53-250	Water: H-3	580 ± 70	630 ± 80	560 ± 250
8-17-83	53-251	Water: Sr-90	11.8 ± 0.5	14 ± 1*	10 ± 0.8
		I-131	13.5 ± 0.8	16 ± 1	17 ± 2.1
		Cs-137	30 ± 8	34.1 ± 8.1	31 ± 7.8
8-17-83	53-252	Water: H-3	1750 ± 60	1890 ± 130	1600 ± 270
11-30-83	53-253	Water: Sr-89	1 ± 2	LLD	LLD
		Sr-90	26.7 ± 6	27.0 ± 1.0	24 ± 5.4
		I-131	19 ± 8	8.0 ± 0.4	11 ± 1.5
		Co-60	30 ± 5	28.9 ± 4.8	35 ± 7.8
		Cs-134	20 ± 4	16.1 ± 4.1	15 ± 8.0
		Cs-137	22 ± 4	23.9 ± 5.0	22 ± 8.4
11-30-83	53-254	Water: H-3	1500 ± 200	1450 ± 100	1630 ± 290

* Based on reanalysis, the first analysis, although in general agreement with the QC Lab appeared to be biased high.

(1) Uncertainties are based on counting statistics and are specified at the 95% confidence interval.

LLD Lower Limit of Detector.

TABLE III.7
QUALITY CONTROL DATAQC Sample Comparisons
(All Analyses in pCi/l)

Sample Date	Ident. No.	Sample Type and Analyses	Independent Lab (1)	DLC Contractor Lab (1)	DLC - QC Lab (1)
2-16-83	52-226	Milk: Sr-89	9 ± 2	11 ± 1	9.2 ± 2.8
		Sr-90	18.6 ± 0.5	18 ± 1	18 ± 2
		I-131	16.1 ± 0.3	12 ± 1	13 ± 2
		Cs-134	17 ± 6	12.5 ± 6.3	11 ± 6
		Cs-137	20 ± 6	21.6 ± 6.5	21 ± 7
5-25-83	52-227	Milk: Sr-89	6 ± 2	5.6 ± 1.8	5.7 ± 2.9
		Sr-90	10.4 ± 0.5	8.6 ± 0.8	9.2 ± 1.5
		I-131	11.4 ± 1.3	12 ± 1	11 ± 1.6
		Cs-134	11 ± 6	11 ± 0.2	8.0 ± 7.4
		Cs-137	21 ± 4	22.2 ± 8.1	22 ± 9.6
8-17-83	52-228	Milk: Sr-89	14 ± 2	16 ± 2	27 ± 11
		Sr-90	21.6 ± 0.5	18 ± 1	14 ± 2.7
		I-131	7 ± 0.7	8.8 ± 0.2	7.4 ± 1.0
		Cs-134	40 ± 5	34 ± 8.1	25 ± 9.0
		Cs-137	48 ± 5	46.6 ± 9.2	39 ± 8.4
11-30-83	52-229	Milk: I-131	27 ± 6	23.0 ± 10	22 ± 2.3
		Cs-134	13 ± 4	7.5 ± 4.1	13 ± 4.6
		Cs-137	15 ± 4	13.0 ± 3.8	11 ± 8.2

(1) Uncertainties are based on counting statistics and are specified at the 95% confidence interval.

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7. Nuclear Regulatory Commission (NRC) Program

The Nuclear Regulatory Commission (NRC) also conducted a surveillance program in the vicinity of the site. Samples of air, river water, drinking water, sediment, milk, vegetation, fish and radiation monitoring are included in their program. The Commonwealth of Pennsylvania's radiological laboratory is utilized by the NRC for analyzing these samples. Comparison of results also indicated agreement between the NRC Laboratory and the Duquesne Light Company Contractor Laboratory.

B. Evaluation of the Quality Control (QC) Program Data

The split sample program indicates that the Contractor laboratory is performing satisfactorily. In addition, three (3) independent laboratories are used to supplement the regular program. Comparisons between the independent laboratories and the Contractor laboratory is acceptable, and demonstrates a satisfactory performance by the DLC contractor.

Based on all available QC data and the data from the Contractor's internal EPA Interlaboratory Cross Check Program, the Environmental Monitoring Program for 1983 is acceptable with respect to both accuracy and measurement.

C. Standard Requirements and Limitations for Radiological and Other Effluents

The Shippingport Atomic Power Station and Beaver Valley Power Station are governed by rules and regulations of the Federal Government and the Commonwealth of Pennsylvania. Effluent releases at both stations are controlled to ensure that limits set by Federal or State governments are not exceeded. In addition, self-imposed limits have been established to further limit discharges to the environment.

Shippingport Atomic Power Station is operated in compliance with regulations and permits involving radioactive and other effluents. Limits noted in Department of Energy (DOE) Orders 5484.1 and 5480.1, Ohio River Valley Water Sanitation Commission (ORSANCO) Standards No. 1-70 and 2-70, Pennsylvania Department of Environmental Resources - Industrial Waste Permit #1832, and Environmental Protection Agency (EPA) National Pollutant Discharge Elimination System (NPDES) Permit #PA-0001589, Pennsylvania Department of Environmental Resources Industrial Waste Permit #0472205, and Pennsylvania Department of Environmental Resources Radioactive Gaseous Discharge Permit are observed and followed.

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Beaver Valley Power Station is subject to regulations which include the Code of Federal Regulations 10 CFR (Energy), Pennsylvania Department of Environmental Resources (PDER) Industrial Waste Permit #0473211, Sewage Treatment Facilities Permit #0479403, Gaseous Discharge Permit #04-306-001, PA Code - Title 24, Part I, Ohio River Valley Water Sanitation Commission (ORSANCO) Standards No. 1-70 and 2-70, Environmental Protection Agency (EPA), National Pollution Discharge Elimination (NPDES) Permit #0025615, and the Beaver Valley Power Station Technical Specifications.

D. Reporting Levels

A report is required to be submitted to the Nuclear Regulatory Commission when the level of radioactivity in an environmental sampling medium exceeds the limits specified in the Beaver Valley Power Station Technical Specifications when averaged over any calendar quarter. Also, when more than one of the radionuclides are detected in the sampling medium, this report shall be submitted if:

$$\frac{\text{Concentration (1)}}{\text{Limit Level (1)}} + \frac{\text{Concentration (2)}}{\text{Limit Level (2)}} + \dots \geq 1.0$$

There were no analytical results of environmental samples during 1983 which exceeded Beaver Valley Power Station reporting levels.

IV. MONITORING EFFLUENTSA. Monitoring of Liquid Effluents

Description of Liquid Effluents at the Shippingport Atomic Power Station and the Beaver Valley Power Station.

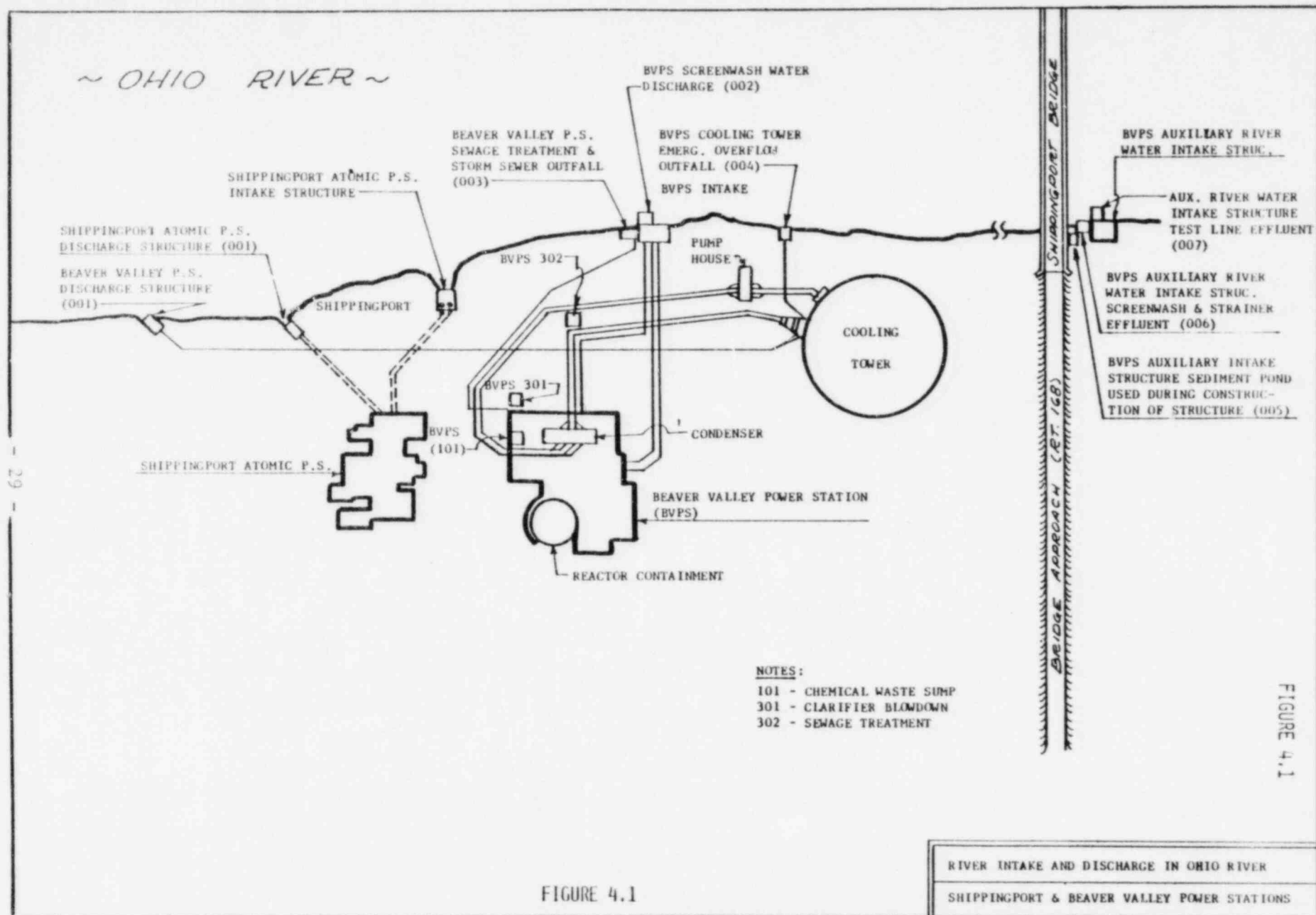
Most of the water required for the operation of the Beaver Valley and Shippingport stations is taken from the Ohio River, and returned to the river, used for makeup to various plant systems, consumed by station personnel, or discharged to a septic system. In addition, small amounts of well water and liquid effluents are discharged to the Ohio River using discharge points shown in Figure 4.1. Figures 4.2 through 4.5 are schematic diagrams of liquid flow paths for Shippingport and Beaver Valley respectively. The following four (4) tables summarize radioactive liquid effluents at both the Shippingport and Beaver Valley Power Stations:

Table IV.A.1 - Effluent Treatment, Sampling, and Analytical Procedures - Shippingport

Table IV.A.2 - Effluent Treatment, Sampling, and Analytical Procedures - Beaver Valley

Table IV.A.3 - Results of Liquid Effluent Discharges to the Environment - Shippingport

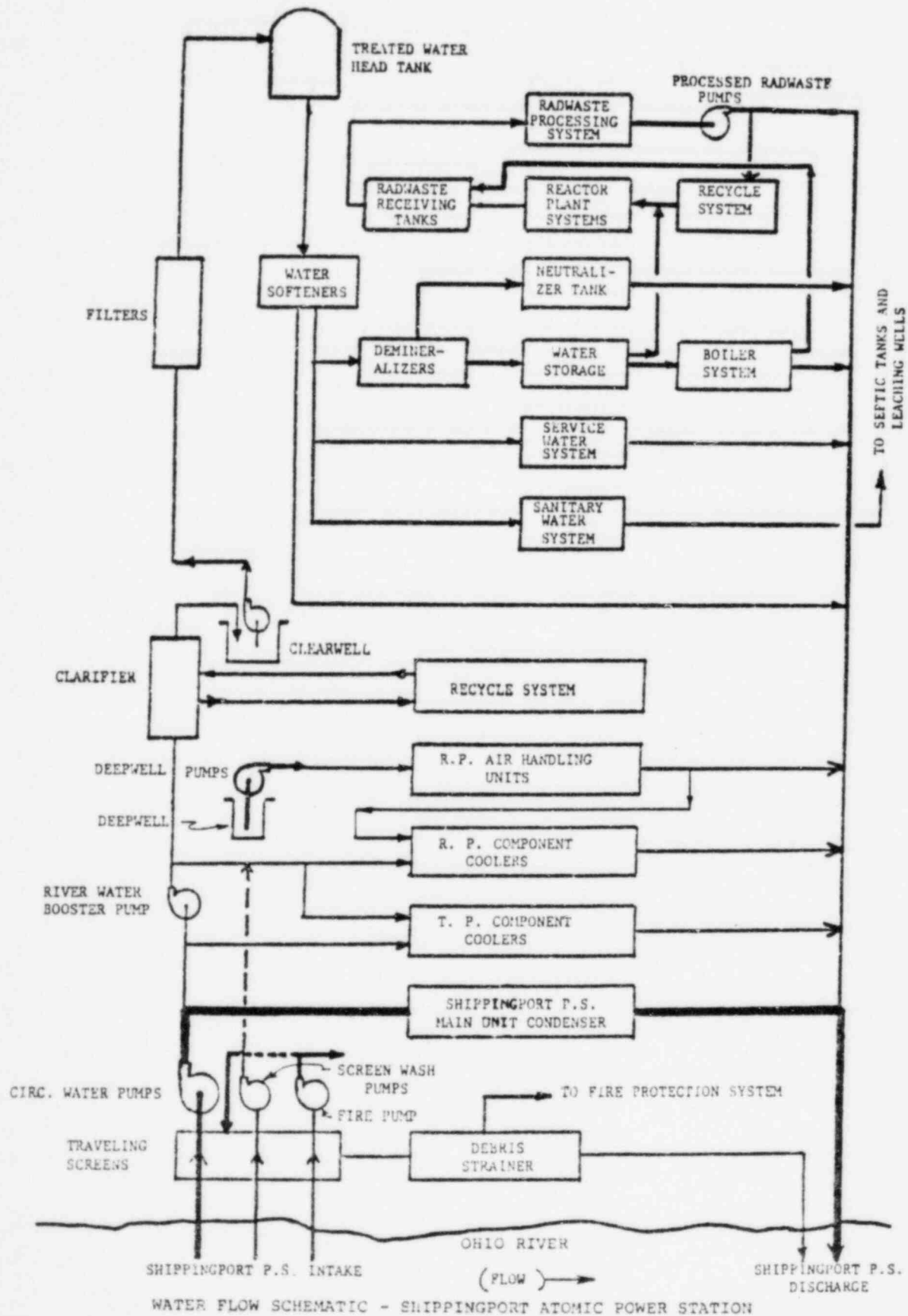
Table IV.A.4 - Results of Liquid Effluent Discharges to the Environment - Beaver Valley



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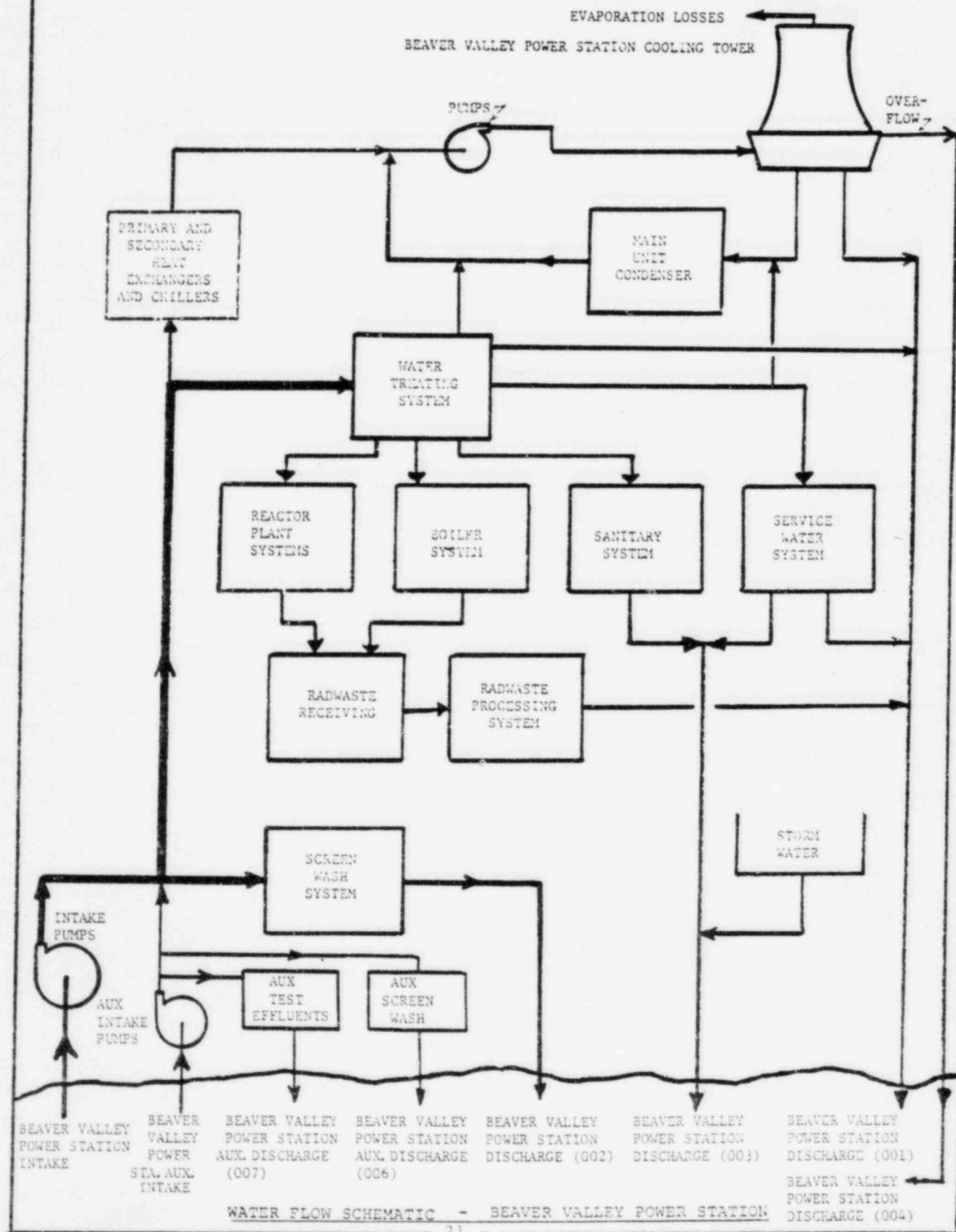
FIGURE 4.2



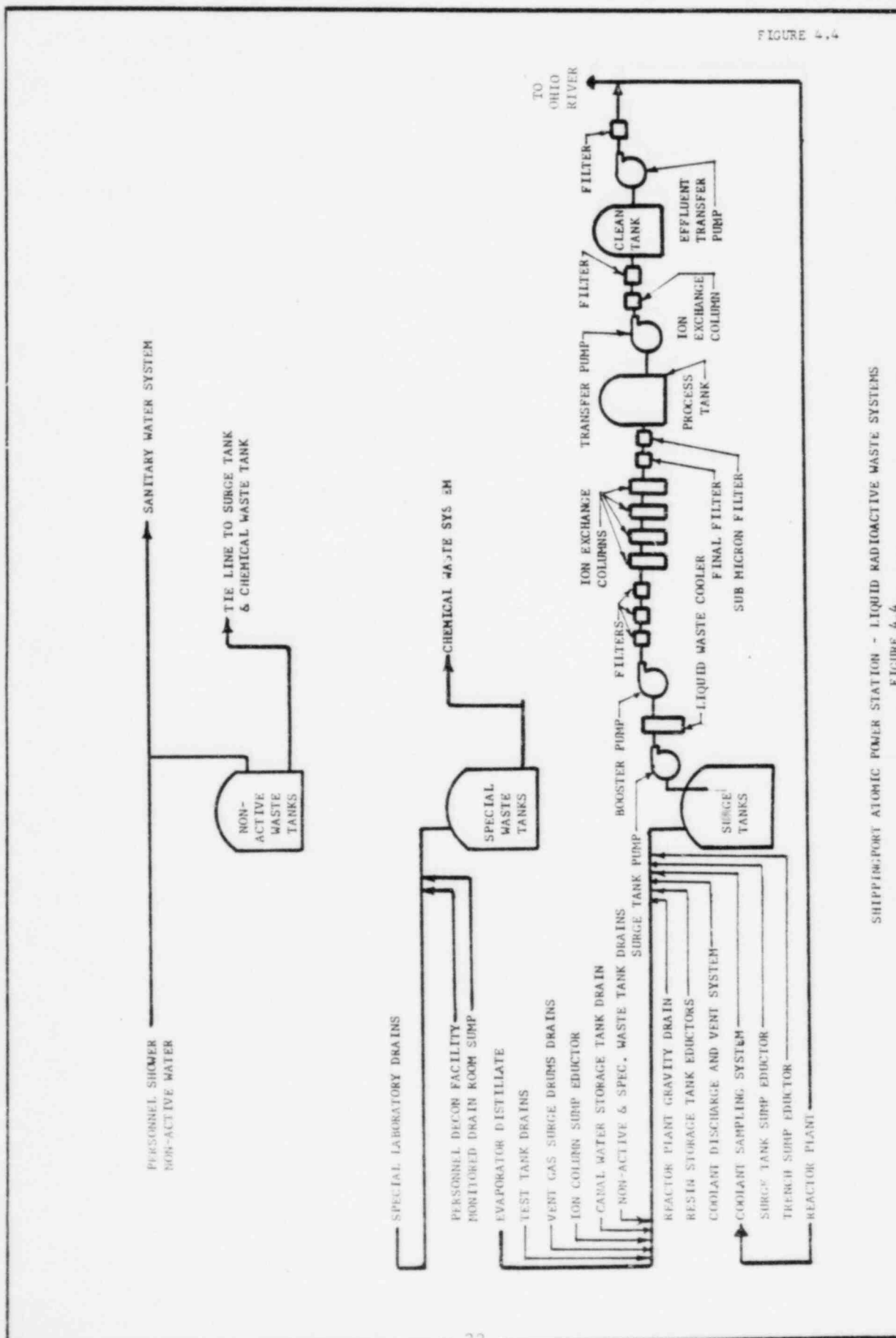
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FIGURE 4.3

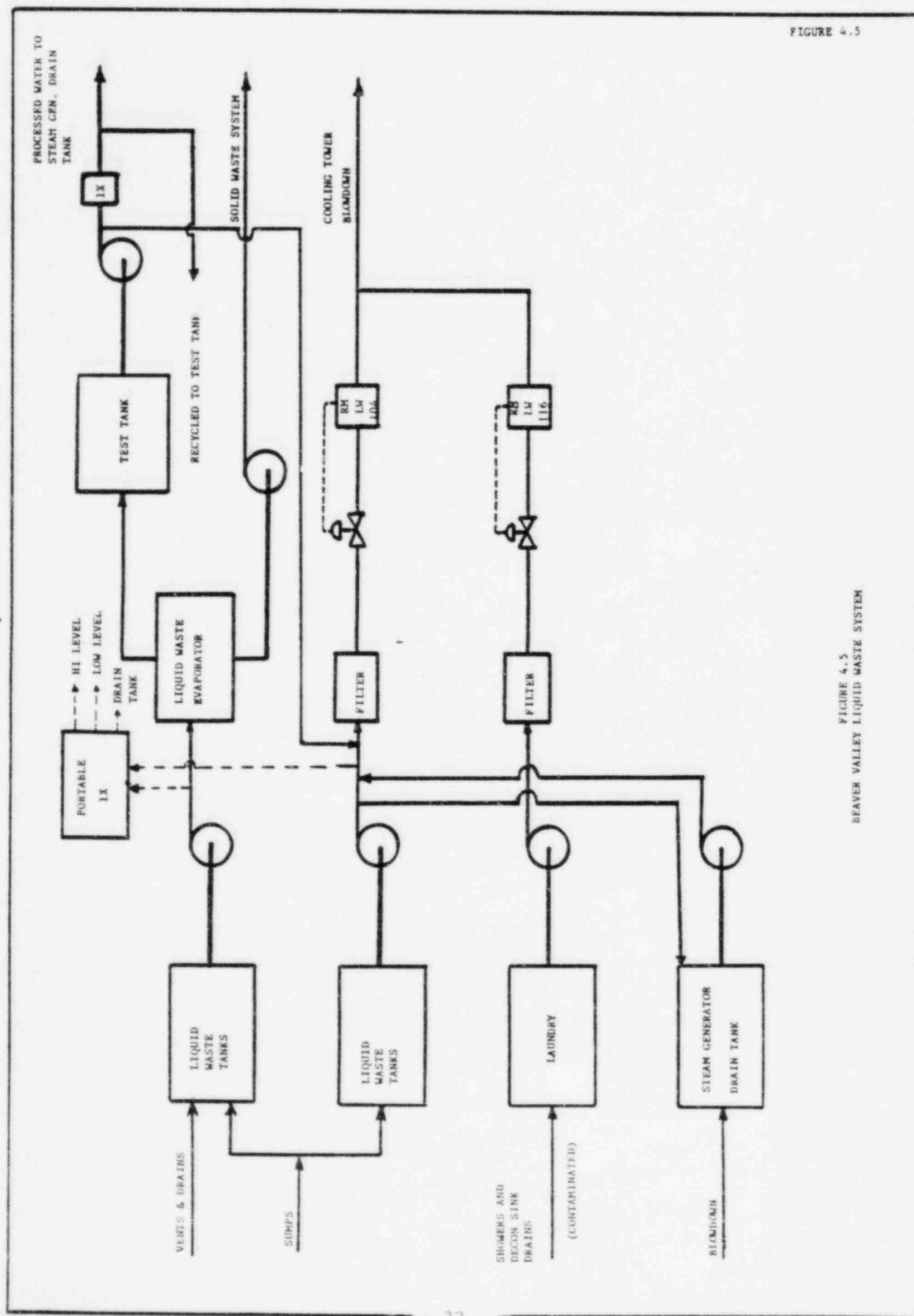


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SHIPPINGPORT ATOMIC POWER STATION - LIQUID RADIOACTIVE WASTE SYSTEMS
FIGURE 4.4

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TABLE IV.A.1

1. Effluent Treatment, Sampling and Analytical Procedures - Shippingport

<u>Effluent Type</u>	<u>Treatment, Sampling and/or Monitoring</u>	<u>Standard and/or Analytical Procedures</u>
(a) Steam System Blowdown	Directed to radwaste system if radioactive. Normally directed to discharge channel where it is diluted by circulating cooling water.	Secondary water is sampled for any radioactive contamination. A 1000 ml sample counted in a multichannel analyzer for 10 minutes for gross activity. The counter can measure a minimum detectable activity (MDA) of 8.4×10^{-8} $\mu\text{Ci/ml}$.
(b) Radioactive Waste Liquids	Collected, segregated and processed as one of two types of liquid wastes: (a) special waste (b) radioactive waste Sample taken of batch before processing to remove radioactivity and reuse in plant systems. See Figure 4.4.	A 3000 ml sample is counted for gross gamma activity. The counter can measure a minimum detectable activity (MDA) of 4×10^{-8} $\mu\text{Ci/ml}$. Also, a 100ml sample is evaporated and counted for gross beta activity. This counter can measure a minimum detectable activity (MDA) of 1×10^{-8} $\mu\text{Ci/ml}$.

TABLE IV.A.2

1. Effluent Treatment, Sampling and Analytical Procedures - Beaver Valley

<u>Effluent Type</u>	<u>Treatment, Sampling and/or Monitoring</u>	<u>Standard and/or Analytical Procedures</u>
(a) Steam System Blowdown	Recycled or directed to Radwaste System for discharge.	If discharged, procedures adhere to Technical Specifications.
(b) Radioactive Waste	Concentration of radioactive materials released in waste effluents shall not exceed values specified in 10 CFR 20, Appendix B, Table II for unrestricted areas, and the Environmental Technical Specifications.	Procedures adhere to requirements of Technical Specifications.

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TABLE IV.A.3

2. Results: Shippingport

<u>Effluent Type</u>	<u>Results for 1983</u>
(a) Steam System Blowdown	Because of final plant shutdown on October 1, 1982, there were no boiler blowdowns during 1983.
(b) Radioactive Waste Liquids	<p>Since Shippingport first went into operation in 1957, the total activity of liquid waste discharged each year has decreased more or less continuously. For example, radioactivity exclusive of tritium decreased from a high of 0.53 Ci in 1965 to a low of less than 0.001 Ci in the years 1974, 1975, 1976, and 1977. There was no radioactive liquid discharged in 1978, 1979, 1980, 1981, or 1982.</p> <p>Discharging of radioactive liquid waste resumed in 1983, with a total activity, exclusive of Tritium, of less than 0.0001 Ci being released. The total Tritium released was approximately 0.672 Ci.</p>

TABLE IV.A.4

2. Results: Beaver Valley

<u>Effluent Type</u>	<u>Results for 1983</u>
(a) Steam System Blowdown	The Steam System Blowdown was recycled or directed to the Radwaste System where it was monitored and discharged in accordance with conditions noted in the Environmental Technical Specifications
(b) Radioactive Waste Liquids	Liquid effluents from the Beaver Valley Power Station were released in accordance with conditions noted in the Environmental Technical Specifications. No limits were exceeded. These values have been reported in the Beaver Valley Power Station Semiannual Effluent Reports for 1983.

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IV. MONITORING EFFLUENTS

B. Monitoring of Airborne Effluents

1. Description of Airborne Effluent Sources

a. Shippingport Atomic Power Station

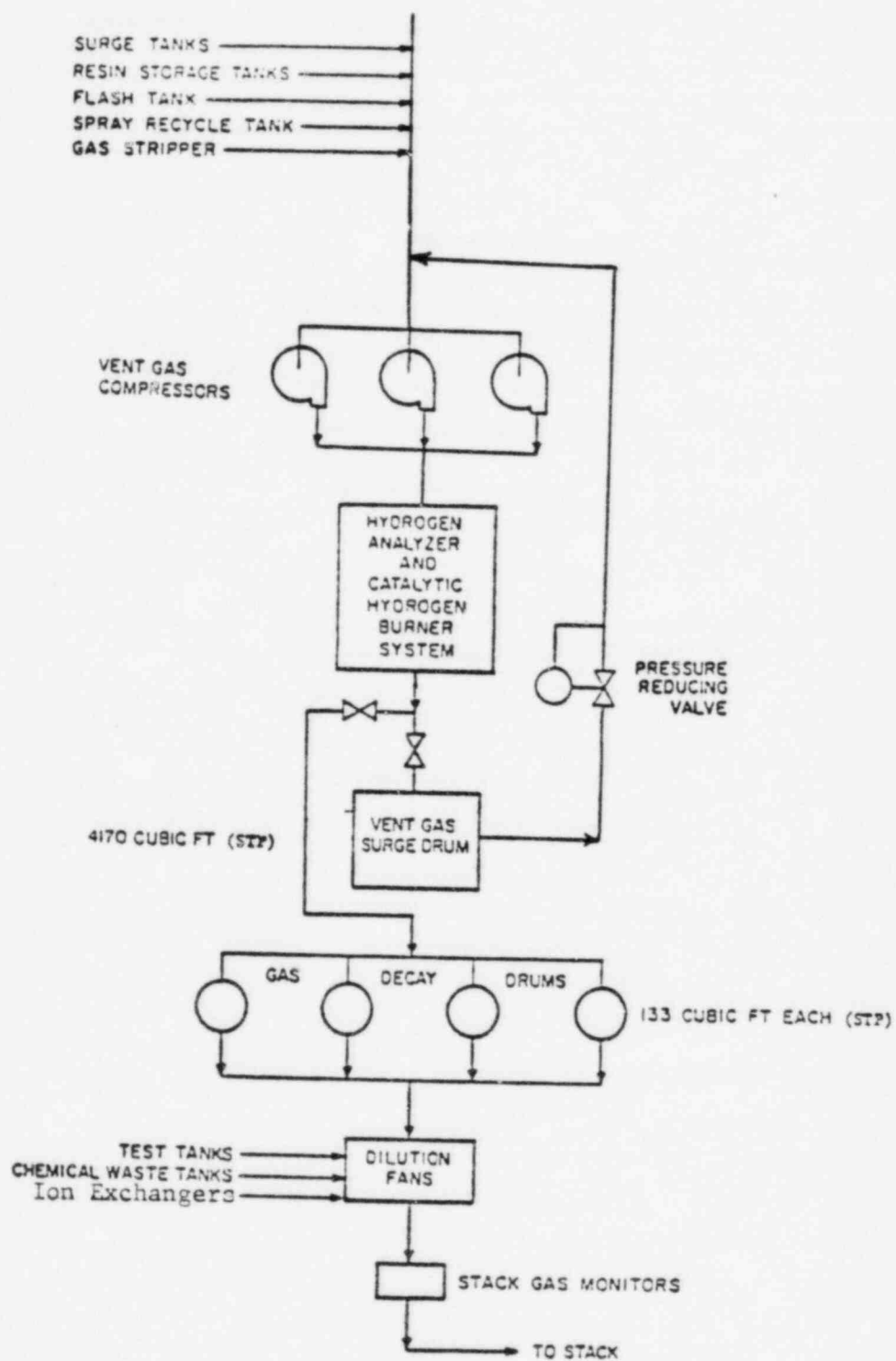
The potential source of airborne radioactivity associated with the Shippingport station is the radioactivity contained in the reactor coolant and canal systems. These systems contain the activated corrosion and wear products, activated impurities in reactor coolant, and small quantities of fission products originating from naturally occurring uranium impurity. This material could become airborne from reactor coolant and canal sampling operations, defueling operation, maintenance and overhaul operations, or working on contaminated components removed from these systems. Stringent radiological controls which have been developed during 26 years of operations at Shippingport are exercised during these operations to prevent radioactivity from becoming airborne. Cobalt-60 is the nuclide of primary concern because of its long radioactive half-life and its concentration in reactor coolant. This radionuclide, present in the form of minute insoluble particles, could become airborne during operations on contaminated components removed from these systems. However, strict radiological surveillance is maintained throughout the plant, including continuous monitoring of airborne radioactivity in the operating spaces to ensure that concentrations are less than the uncontrolled area limits specified in DOE Order 5484.1. In addition, air exhausted from potentially contaminated areas, such as decontamination and maintenance areas, is passed through high-efficiency particulate air filters. These filters are routinely serviced, changed, and tested in-place.

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The principle environmental release points for the Shippingport Atomic Power Station are the containment ventilation exhaust and the Radioactive Waste Processing System exhaust. These points are continuously monitored, and analyses are performed on charcoal cartridges weekly for I-131 and monthly for I-133 and I-135. Additionally, weekly continuous air samples are obtained on fixed filter papers which are analyzed weekly for gross beta, and composited monthly to identify gamma emitting isotopes. Composite of the particulate filters are also analyzed monthly for gross alpha determinations and quarterly for Sr-89 and Sr-90. A monthly gas sample is also obtained and analyzed for tritium for each release point.

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Shippingport Atomic Power Station Cesium
Radioactive Waste Processing System

FIGURE 4.6

IV. MONITORING EFFLUENTS

a. Shippingport Atomic Power Station (continued)

Reactor plant exhausts from the Decontamination Room, Sample Preparation Room, Radiochemistry Laboratory, Gaseous Waste System, Compacting Station, and Decontamination Waste Drain Receiver Tank are continuously sampled with fixed filter samplers. These samples are analyzed weekly for gross beta, and composited monthly to identify gamma emitting isotopes.

Processing of noble gases (predominantly short lived Xe-133) is accomplished by collecting and storing the gases in Shippingport RWP vent gas system. After sampling and analysis, the gases are released when the storage tanks are full. Figure 4.6 shows a schematic diagram of the gaseous waste system in the radioactive waste disposal system at Shippingport.

b. Beaver Valley Power Station (BVPS)

The Beaver Valley Power Station identifies isotopes according to the Environmental Technical Specifications and Regulatory Guide 1.21. Prior to waste gas decay tank batch releases and containment purge releases, an analysis of the principal gamma emitters is performed. The principle gamma emitters include noble gases, iodines, and particulates. Figure 4.7 shows the gaseous radwaste system at Beaver Valley Power Station.

The environmental gaseous release points also require specific nuclide identification. These points include the Process Vent located on top of the Cooling Tower, the Ventilation Vent located on the top of the Auxiliary Building, and Supplementary Leak Collection and Release System (SLCRS) Vent located on top of the Containment. These points are continuously monitored. Principal gamma emitters and tritium are analyzed on a monthly basis. Analysis is also done on charcoal cartridges for I-131, I-133, and I-135 that have continuously sampled the gas stream for a week. Weekly continuous samples are also obtained on filter paper to identify particulate gamma emitting isotopes. Composites of the particulate samples are analyzed monthly for gross alpha determinations and monthly for Sr-89 and Sr-90.

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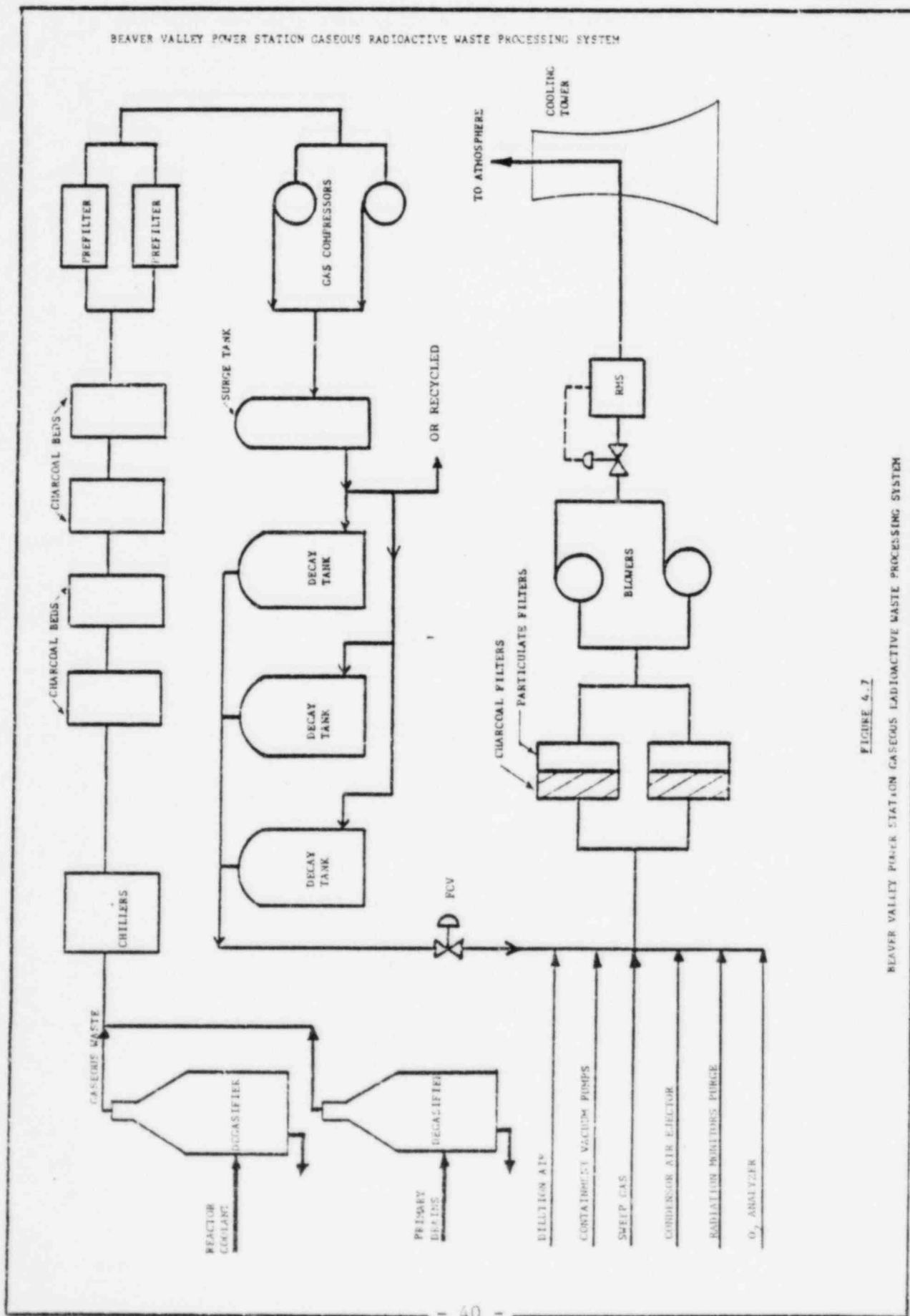


FIGURE 4.7
BEAVER VALLEY POWER STATION GASEOUS RADIOACTIVE WASTE PROCESSING SYSTEM

IV. MONITORING EFFLUENTS2. Airborne Effluent Treatment and Sampling

a. Shippingport Atomic Power Station

Gaseous wastes stripped from the reactor coolant at the Shippingport Station are circulated through a hydrogen analyzer and catalytic hydrogen burner system where the hydrogen is removed. The gases are initially stored in a vent gas surge drum, and subsequently compressed and transferred to one of four gas storage drums. The decayed gases are sampled prior to release. In addition, the exhaust from the containment is equipped with high efficiency particulate air filters and monitoring devices to prevent releases of radioactive particulates. Protective devices are utilized in the event of high airborne activity to automatically seal off the primary containment to prevent an inadvertent release of radioactivity. Reactor plant exhausts from the Decontamination Room, Sample Preparation Room, Radiochemistry Laboratory, Compacting Station, and Decontamination Waste Drain Receiver Tank are also equipped with high efficiency particulate air filters, and are continuously monitored for radioactive particulates by the use of fixed filter monitors. Exhausts from the Gaseous Waste System are filtered and sampled for radioactivity at the release point also. Continuous air monitors are located within the containers, and other plant areas to constantly monitor the condition of the air. A stack release diagram is shown in Figure 4.8 identifying ventilation and gaseous release points for the Shippingport Atomic Power Station.

b. Beaver Valley Power Station

Radioactive gases enter the gaseous waste disposal system from the degasifier vent chiller of the boron recovery system, and are directed to the gaseous waste charcoal delay subsystem upstream of the overhead gas compressor where the gas is chilled to condense most of the water vapor. Gases from the degasifier vent chillers contain primarily hydrogen and water vapor. A small amount of nitrogen and traces of the radioisotopes xenon, krypton, cobalt, cesium, manganese and iodine are also present in the gaseous effluent.

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DISCHARGE POINTS - GASEOUS WASTES

FIGURE 4.8

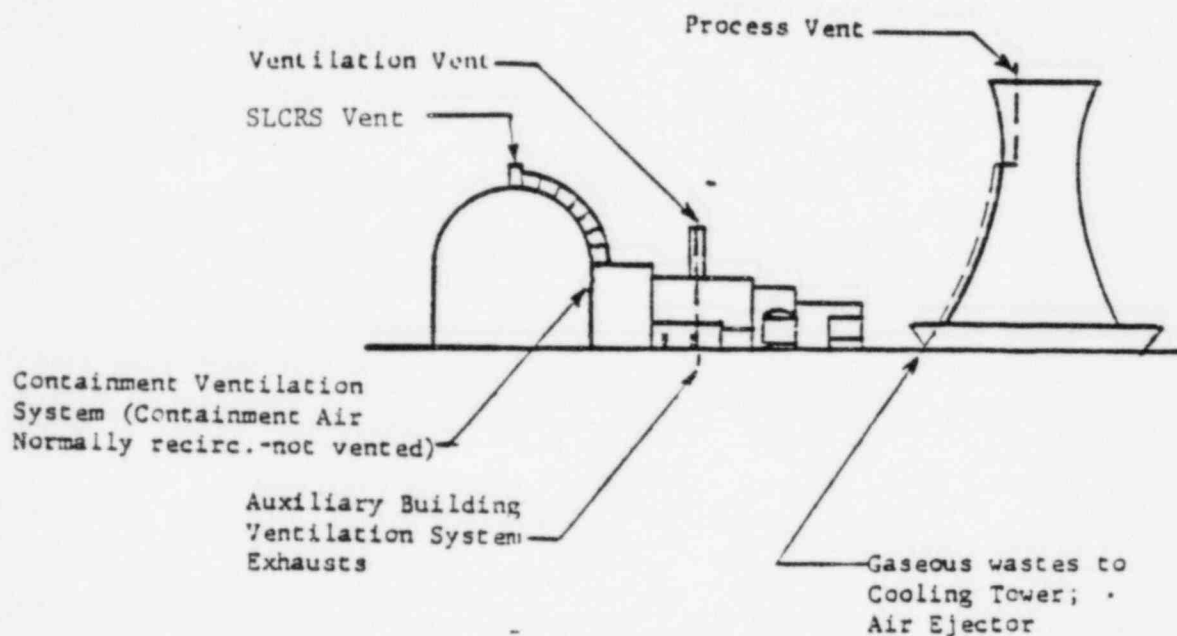
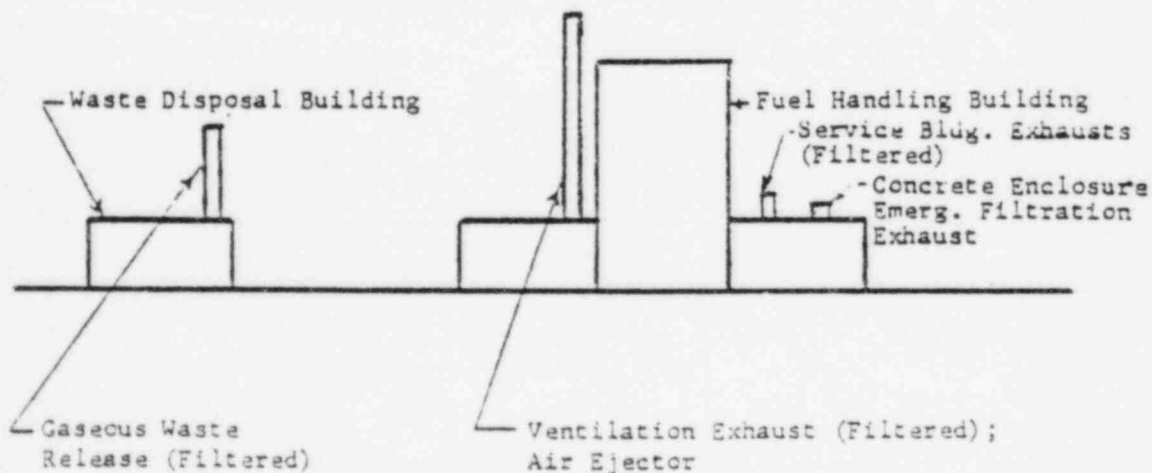
GASEOUS RELEASE POINTS - BEAVER VALLEY POWER STATIONGASEOUS RELEASE POINTS - SHIPPINGPORT ATOMIC POWER STATION

FIGURE 4.8

IV. MONITORING EFFLUENTS

b. Beaver Valley Power Station (continued)

The overhead gas compressor directs the radioactive gas stream to a gas surge tank. The system is designed to return most of the gas to the volume control tank in the Chemical and Volume Control System (CVC System). A quantity of gas is periodically discharged from the surge tank to one of the three (3) decay tanks for eventual release to the atmosphere via the process vent on top of the cooling tower. After the decay tanks are sampled and authorization obtained for discharge, the flow of the waste gases from the decay tanks (2 scfm) is recorded and rapidly diluted with about 1000 scfm of air in order to limit hydrogen concentration. The gases are then combined with the containment vacuum system exhaust, aerated vents of the vent and drain system, and the main air ejector effluent. The mixture is then filtered through one of the gaseous waste disposal filters, each of which consists of a charcoal bed and a high efficiency filter. The filtered gases are then discharged by one of the gaseous waste disposal blowers to the atmosphere via the process vent on the top of the cooling tower. The radioactivity levels of the stream are monitored continuously. Samples are also taken to determine the rate of activity released to the atmosphere. Should the radioactivity release concentration of the stream go above the allowable setpoint, a signal from the radiation monitor will stop all flow from the decay tanks.

During a shutdown period after the containment has been sampled and the activity levels determined, the containment may be purged through the Ventilation Vent located on top of the Auxiliary Building or the Supplementary Leak Collection and Release System (SLCRS) Vent located on top of the Reactor Containment Building or the Process Vent located on top of the Cooling Tower.

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Areas in the Auxiliary Building subject to radioactive contamination are monitored for radioactivity prior to entering the common ventilation vent. These individual radiation monitors aid in identifying any sources of contaminated air. The ventilation vent is also monitored continuously and sampled periodically. Upon a high activity alarm, automatic dampers divert the system's exhaust air stream through one of the main filter banks in the supplementary leak collection and release system (SLCRS) and to the SLCRS Vent. Release points are shown in Figure 4.8 for the Beaver Valley Power Station.

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DUQUESNE LIGHT COMPANY
1983 Annual Radiological Environmental ReportIV. MONITORING EFFLUENTS

b. Beaver Valley Power Station (continued)

Each filter bank consists of roughing filters, charcoal filters, and pleated glass fiber type HEPA filters. The roughing filters remove large particulates to prevent excessive pressure drop buildup on the charcoal and HEPA filters. The charcoal filters are effective for radioactive iodine removal and the HEPA filters remove particulates and charcoal fines.

3. Analytical Procedures for Sampling Airborne Effluents

a. Shippingport Atomic Power Station

The following tabulates the gaseous sampling and analysis schedule:

Vent Gas Decay Tank Releases

Sample Type	Sampling Frequency	Type Of Activity Analysis	Detectable Concentration
Gas From Decay Tank	Prior to Discharge	Gamma Ray Spectrum of Gas Sample *	3×10^{-7} $\mu\text{Ci/cc}$
Gas from Decay Tank	Prior to Discharge	H-3	2×10^{-7} $\mu\text{Ci/cc}$
Gas from Decay Tank	Prior to Discharge	C-14	1×10^{-7} $\mu\text{Ci/cc}$

- * A gas sample of measured volume is counted in a multi-channel analyzer for 10 minutes for gross activity. The counter has a minimum detectable activity (MDA) of 3×10^{-7} $\mu\text{Ci/cc}$ for the predominant nuclide of Xe-133.

IV. MONITORING EFFLUENTS

3. Analytical Procedures for Sampling Airborne Effluents (continued)

Air Exhausts

Sample Type	Sampling Frequency	Type Of Activity Analysis	Detectable Limits
(1) Air from the Plant Ventilation Exhaust Effluent Stream	Continuous	Gross Gamma	1.2×10^{-6} $\mu\text{Ci/cc}$ (Gas, Channel 8 ORMS)* 5×10^{-10} $\mu\text{Ci/cc}$ (Particulate; Channel 12 ORMS)*
(2) Particulate Filter in Plant Ventilation Effluent Stream and Gaseous Waste System Exhaust	Continuous	Gross Beta Weekly Sample	1×10^{-14} $\mu\text{Ci/cc}$ (Particulate)
Particulate Filter in Plant Ventilation Effluent Stream and Gaseous Waste System Exhaust	Continuous	Principle Gamma Monthly Composite (Weekly if Gross Beta $\geq 1 \times 10^{-13}$)	1×10^{-13} $\mu\text{Ci/cc}$ Particulate
Particulate Filter in Plant Ventilation Effluent Stream and Gaseous Waste System Exhaust	Continuous	Gross Alpha Monthly	2×10^{-15} $\mu\text{Ci/cc}$ Particulate
Particulate Filter in Plant Ventilation Effluent Stream and Gaseous Waste System Exhaust	Continuous	Sr-89, Sr-90 Quarterly	5×10^{-14} $\mu\text{Ci/ml}$ (Sr-89) 1×10^{-14} $\mu\text{Ci/ml}$ (Sr-90) Particulate
(3) Charcoal Cartridge in Plant Ventilation Effluent Stream and Gaseous Waste System Exhaust	Continuous	I-131 Weekly	1×10^{-13} $\mu\text{Ci/ml}$
Charcoal Cartridge in Plant Ventilation Effluent Stream and Gaseous Waste System Exhaust	Monthly	I-133, I-135 Monthly	1×10^{-13} $\mu\text{Ci/ml}$ (I-133) 1×10^{-12} $\mu\text{Ci/ml}$ (I-135)
(4) Evacuated Bomb Sample In Plant Ventilation Effluent Stream and Gaseous Waste System Exhaust	Monthly	H^3	2×10^{-7} $\mu\text{Ci/ml}$
(5) Particulate Filter in Reactor Plant Exhaust from Decontamination Room, Sample Preparation Room, Radiochemistry Laboratory, Compacting Station, and Decontamination Waste Drain Receiver Tank.	Continuous	Gross Beta Weekly	1×10^{-14} $\mu\text{Ci/cc}$ Particulate
Particulate filter in Reactor Plant Exhaust from Decontamination Room, Sample Preparation Room, Radiochemistry Laboratory, Compacting Station, and Decontamination Waste Drain Receiver Tank.	Continuous	Principal Gamma Monthly Composite (Weekly if Gross Beta $\geq 1 \times 10^{-13}$)	1×10^{-13} $\mu\text{Ci/cc}$ Particulate

ORMS - Operational Radiation Monitoring System

* Although the ORMS Channels have no specific function as far as effluent monitoring and reporting is concerned, these two (2) channels are being listed for information purposes. It is also noted that these channels provide alarm functions in the Main Control Room when levels of 1.2×10^{-6} $\mu\text{Ci/cc}$ are reached on Channel 8, or 1×10^{-9} $\mu\text{Ci/cc}$ on Channel 12. Additionally, they shut the ventilation system butterfly valves when levels of 1.2×10^{-6} $\mu\text{Ci/cc}$ are reached on Channel 8 or 1×10^{-9} $\mu\text{Ci/cc}$ on Channel 12.

IV. MONITORING EFFLUENTS3. Analytical Procedures for Sampling Airborne Effluents (continued)

b. Beaver Valley Power Station

The following tabulates the gaseous sampling and analysis schedule:

	Gaseous Source	Sampling Frequency	Type Of Activity Analysis	Detectable Concentrations ($\mu\text{Ci}/\text{ml}$) ^a
(1)	Waste Gas Decay Tank Releases	Each Tank	Principal Gamma Emitters	10^{-4b}
			H-3	10^{-6}
(2)	Containment Purge Releases	Each Purge	Principal Gamma Emitters	10^{-4c}
			H-3	10^{-6}
(3)	Environmental Release Points	Monthly (Gas Samples)	Principal Gamma Emitters	$10^{-4b,c}$
			H-3	10^{-6}
		Weekly (Charcoal Sample)	I-131	10^{-12}
		Weekly (Charcoal Sample)	I-133, I-135	10^{-10}
		Weekly ^d (Particulates)	Principal Gamma Emitters (Ba-La-140, I-131, and others)	10^{-11}
		Monthly ^d Composite (Particulates)	Gross α	10^{-11}
		Quarterly ^d Composite (Particulates)	Sr-90 and Sr-89	10^{-11}

^a The above detectability limits for activity analysis are based on technical feasibility and on the potential significance in the environment of the quantities released. For some nuclides lower detection limits may be readily achievable, and when nuclides are measured below the stated limits, they should also be reported.

^b For certain mixtures of gamma emitters, it may not be possible to measure radionuclides at levels near their sensitivity limits when other nuclides are present in the sample at much higher levels. Under these circumstances, it will be more appropriate to calculate the levels of such radionuclides using observed ratios with those radionuclides which are measured.

^c Analyses shall also be performed following each refueling, startup, or similar operational occurrence which could alter the mixture of nuclides.

^d To be representative of the average quantities and concentrations of radioactive materials in particulate form released in gaseous effluents, samples should be collected in proportion to the rate of flow of the effluent stream.

IV. MONITORING EFFLUENTS4. Results

a. Shippingport Atomic Power Station

Analyses for the particulate airborne radioactivity in the plant effluents indicated that the gross alpha and beta activity concentrations were at or very near the Lower Limit of Detection (LLD). Analytical results of charcoal filter samples showed that there were no instances of radioiodine concentrations above LLD.

Specific gamma analyses of weekly and monthly composite air filter samples were also performed. Results showed naturally occurring radioactivity typical of "background air" and nuclides attributable to worldwide fallout from nuclear weapons testing.

During 1983, there were six (6) instances where activity attributable to plant operations was measured. In each of these instances, Cobalt-60 activity was measured in the Decontamination Room Waste Receiver Exhaust. The total amount of Cobalt-60 activity released was 0.000000008330 Ci with concentrations ranging from $2.10 \times 10^{-15} \mu\text{Ci/cc}$ to $1.51 \times 10^{-13} \mu\text{Ci/cc}$. These concentrations are all well below Federal limits.

Radiostrontium analyses of quarterly composite air filter samples showed Sr-89 and Sr-90 concentrations in air which were at or near the minimum detectable concentrations of $5 \times 10^{-14} \mu\text{Ci/ml}$, and $1 \times 10^{-14} \mu\text{Ci/ml}$, respectively. The levels of Sr-89 and Sr-90 observed were extremely low and are typical of "background air" radiostrontium levels. Also, Carbon-14 activities in gaseous effluent releases were estimated based on analysis of primary coolant and analysis of Radioactive Waste Processing System vent gas and tritium activities in gaseous effluents were estimated based on canal water tritium levels. These activity levels were consistent with past reported levels considering plant operations during 1983.

There were five (5) releases of gaseous radioactivity from the Shippingport Atomic Power Station during 1983. The total gaseous radioactivity released from the Shippingport Atomic

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Power Station during 1983 was approximately 0.0000205 curies of Xe-133. The amount of radioactivity released from the Shippingport Atomic Power Station during 1983 was extremely small and had a negligible effect on the environment as shown in Section V.I.6.

b. Beaver Valley Power Station

Gaseous effluents from the Beaver Valley Power Station were released in accordance with conditions noted in the Environmental Technical Specifications. No limits were exceeded. These values have been reported in the Beaver Valley Power Station Semi-Annual Effluent Reports for 1983.

IV. MONITORING EFFLUENTSC. Solid Waste Disposal at the Shippingport and Beaver Valley Power Stations

During the Shippingport Atomic Power Station defueling efforts and Beaver Valley Power Station normal operations and periodic maintenance, small quantities of solid radioactive waste materials were generated such as evaporator concentrates, contaminated rags, paper, plastics, filters, spent ion-exchange resins, and miscellaneous tools and equipment. These were disposed of as solid radioactive waste.

At Shippingport Atomic Power Station and the Beaver Valley Power Station, the compactable wastes were segregated and compressed in a 55-gallon drum to minimize disposal volumes. The compressed waste, plus other drums of noncompactable waste, were then shipped offsite for disposal at a site owned by the Department of Energy or a commercial radioactive material burial site licensed by the Nuclear Regulatory Commission (NRC) or a state under agreement with the NRC. No radioactive waste material was buried at the Shippingport or Beaver Valley Power Station site.

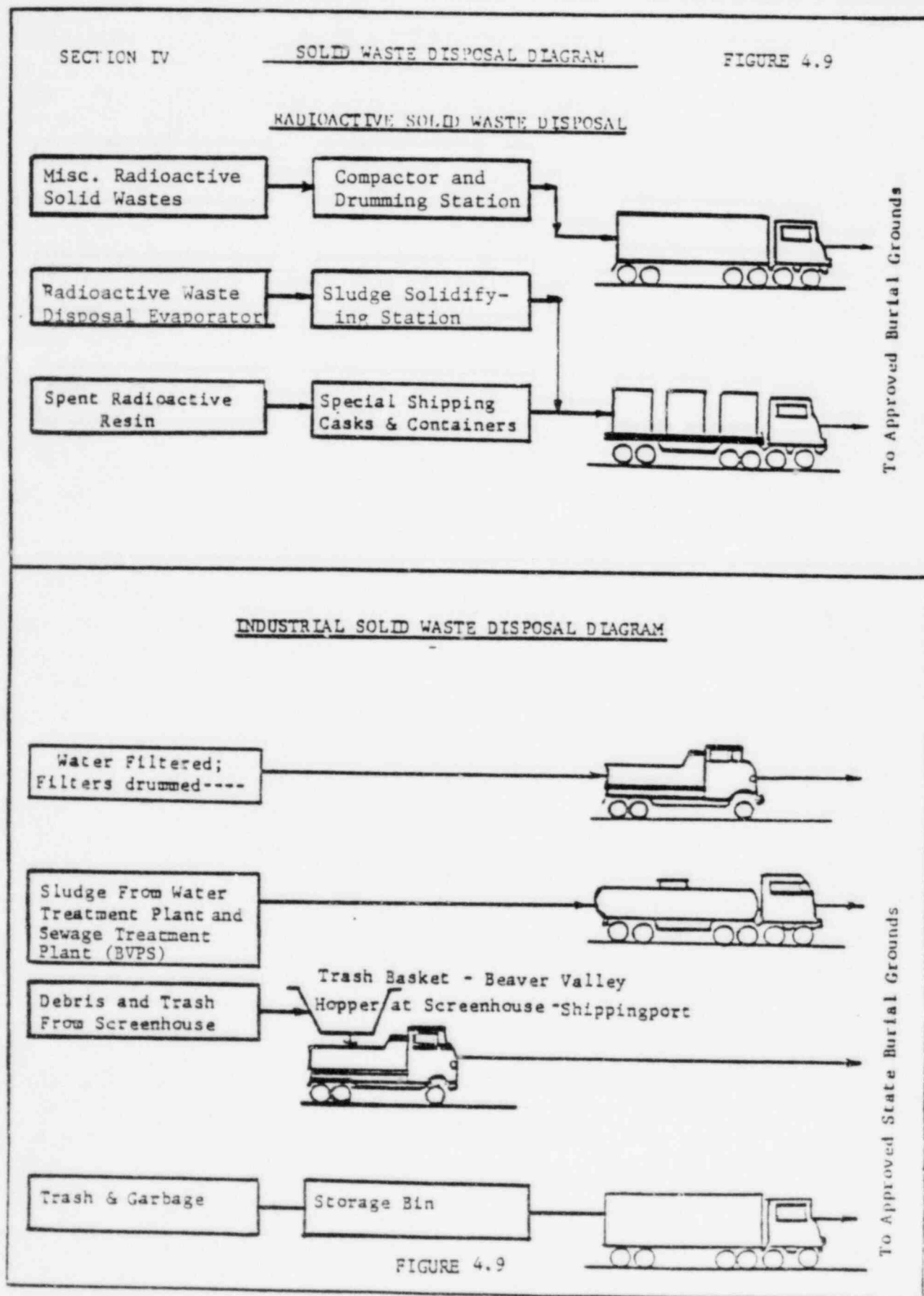
All containers used for packaging, transport, and disposal of radioactive materials met the requirements of the United States Department of Transportation and the Nuclear Regulatory Commission. Shipments offsite were made in accordance with Department of Transportation regulations. Figure 4.9 depicts solid waste handling at the site.

In 1983, the Shippingport plant disposed of a total of 14,130.5 cubic feet of radioactive solid waste having a total radioactivity of about 15.334 curies. This included twenty (20) shipments of low level wastes.

At Beaver Valley Power Station approximately 7,417 cubic feet of radioactive solid waste was shipped offsite in 1983. The actual burial volume of solid waste is slightly higher due to container size (8127 cubic feet). The thirty (30) shipments contained a total activity of 475.1 curies.

Industrial solid wastes from both plants were collected in portable bins, and removed to an approved offsite burial ground. No burning or burial of wastes was conducted at either the Beaver Valley or Shippingport plant.

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V. ENVIRONMENTAL MONITORINGA. Environmental Radioactivity Monitoring Program1. Program Description

The program consists of monitoring water, air, soil, river bottoms, vegetation and foodcrops, cow's milk, ambient radiation levels in areas surrounding the site, and aquatic life as summarized in Table V.A.1. Further description of each portion of the program (Sampling Methods of Sample Analysis, Discussion and Results) are included in parts V-B through V-I of this report.

V-B - Air Monitoring

V-C - Sediments and Soils Monitoring

V-D - Vegetation and Foodcrops

V-E - Cow's Milk

V-F - Environmental Radiation Monitoring

V-G - Fish

V-H - Surface, Drinking and Well Waters

V-I - Estimates of Radiation Dose to Man

TABLE V.A.1
CONSOLIDATED RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM

Type of Sample	DLC Sample Points	Sector	Miles	Sample Point Description	Sample Frequency	Sample Preparation	Analysis Frequency	Analysis ^(b)
1. Air Particulate and Radioiodine	13.	11	1.6	Meyer's Farm	Continuous Sampling with sample collection at least weekly	Weekly Composite ^(d)		Gross Beta, ^(c) I-131
	30	4	0.6	Shippingport, PA. (S.S.)		Monthly Composite ^(d)		Gamma -scan
	46.1	3	2.4	Industry, PA (Tire Co.)				
	32	15	0.8	Midland, PA (S.S.)		Quarterly Composite ^(d)		Sr-89,90
	48(a)	10	16.5	Weirton, WV (a)				
	51	5	8.0	Aliquippa, PA (S.S.)				
	47	14	4.8	East Liverpool, OH				
	27	7	6.2	Brunton's Farm				
	28	1	8.7	Sherman's Farm				
	29B	3	8.1	Beaver County Hospital				
2. Direct Radiation	30	4	0.6	Shippingport, PA (S.S.)	Continuous (TLD)	Quarterly ^(k)		Gamma-Dose
	13	11	1.6	Meyer's Farm		Annually ^(k)		
	46	3	2.5	Industry, PA (Church)				
	32	15	0.8	Midland, PA (S.S.)				
	48 (a)	10	16.5	Weirton, WV (a)				
	45.1	6	2.9	Raccoon Twp, PA Kennedy's Crnrs				
	51	5	8.0	Aliquippa, PA (S.S.)				
	47	14	4.8	East Liverpool, OH				
	70	1	3.0	West. Bvr. School				
	80	9	8.4	Raccoon Park				
	81	9	3.9	Southside School				
	82	9	7.1	Handover Municipal Bldg.				
	83	10	4.5	Mill Creek Rd				
	14	11	2.5	Hookstown				
	84	11	8.5	Nuncor Co. Children Home				
	85	12	5.8	Rts. B & 30 Intersection				
	86	13	5.5	E. Liverpool Cahills House				
	92	12	3.0	Georgetown Rd.				
	87	14	7.0	Calcutta Road				
	88	15	3.1	Midland Heights				
	89	15	4.7	Ohioville				
	90	16	6.2	Fairview School				
	10	4	0.8	Shippingport Boro, PA				
	45	5	2.2	Mt. Pleasant Church				
	60	13	3.1	Haney's Farm				
	93	16	1.3	Sunset Hills, Midland				
	55	10	2.4	McCleary Rd, Hollie Williams				

S.S. - Substation

TABLE V.A.1
CONSOLIDATED RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM
(Continued)

Type of Sample	DLC Sample Points	Sector	Miles	Sample Point Description	Sample Frequency	Sample Preparation	Analysis Frequency	Analysis (b)
2. Direct Radiation (Continued)	28 71 72 298 73 74 75 76 77 59 78 27 79 15 46.1 91 94	1 2 3 3 4 4 5 6 6 7 7 7 8 14 3 2 8	8.7 5.6 3.2 8.1 2.2 6.8 4.3 3.8 5.8 1.1 2.3 6.2 4.6 3.3 2.1 3.7 2.4	Sherman's Farm Brighton Twp. School Logan School Beaver County Hospital Potter Twp. School Comm. Col-Center Twp. Holt Road Raccoon Twp. School Green Garden Rd (Wayne's) Irons Raccoon Mun. Bldg. Brunton's Farm Rt. 18 & Rt. 151 Georgetown Industry PA Tire Co. Pine Grove Rd and Doyle Rd McCleary Rd, Wilson	Continuous (TLD)	Quarterly (k) Annually (k)		Gamma-Dose
3. Surface Water	49.1 2.1 3 49 (a) 2A 5	4 14 13 3 13 14	5.0 1.3 0.2 3.2 0.2 4.8	Arco Polymers (a) Downstream (Midland)Crucible Shippingport Atomic Power Station Discharge Montgomery Dam (Upstream) Downstream BVPS Outfall East Liverpool (raw water)	Intermittent Composite Samples (j) Collected Weekly Weekly Grab Samples Only Daily Grab Sample Only - Collected Weekly (j)	Monthly Composite of Weekly Sample (d) Quarterly Composite		Gross Beta Gross Alpha Gamma-scan Co-60, H-3 Sr-89, Sr-90
4. Groundwater	13 14 15 11	11 11 15 3	1.6 2.6 3.3 0.8	Meyer's Farm Hookstown, PA Georgetown, PA Shippingport Boro	Quarterly	Quarterly		Gamma-scan, Gross Beta, Gross Alpha, H-3
5. Drinking	4 5	14 14	1.3 4.8	Midland, PA (Midland Water Treatment Plant) East Liverpool, OH (East Liverpool Water Treatment Plant)	Intermittent (e) Sample Collected Weekly	Weekly Composite of Daily Sample (d) Monthly Composite (d) Quarterly Composite (d)		Gamma-scan, I-131 Gross Alpha, Gross Beta H-3, Co-60, Sr-89, 90

TABLE V.A.1
CONSOLIDATED RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM
(Continued)

Type of Sample	DLC Sample Points	Sector	Miles	Sample Point Description	Sample Frequency	Sample Preparation	Analysis Frequency	Analysis ^(b)
6. Shoreline Sediment	2A	13	0.2	Downstream BVPS Outfall	Semiannual	Semiannual		Gamma-scan, Gross Beta Gross Alpha Uranium Isotopic Sr-89, 90
	3	13	0.2	Vicinity SAPS Discharge				
	49	3	3.2	Upstream Side of Montgomery Dam (a)				
	50	13	8.2	Upstream side of New Cumberland Dam				
7. Milk	25	10	2.1	Searight's Dairy	Weekly ^(f)	Weekly sample from Searight's only		I-131
	61*	12	3.2	Allison	Biweekly ^(g)	Biweekly (grazing)		Gamma-scan Sr-89, 90 I-131, Cs-137
	69*	7	3.6	Collins	When animals are on pasture; monthly at other times.	Monthly (indoors)		
	96(a)	10	10.3	Windsheimer				
	98*	5	2.9	Foxall	Monthly			Gamma-scan Sr-89, 90 I-131, Cs-137
	100*	14	2.8	Doughty				
	27	7	6.2	Brunton's Dairy (h)		Monthly		
8. Fish	2A	13	0.2	Vicinity of BVPS #1 Station Discharge and Shippingport Dis. Sta.	Semiannual	Composite of edible parts by species (i)		Gamma-scan on edible portions
	49(a)	3	4.7	Upstream Side of Montgomery Dam				
9. Food Crops (Shipp.) (Georg.) (Indus.)	10	4	0.8	(Three locations within 5 miles Selected by Company) Weirton, WV	Annual at harvest if available	Composite of each sample species		Gamma-scan I-131 on green leafy vegetables
	15	14	3.3					
	46	3	2.5					
	48(a)	10	16.5					
10. Feedstuff and Summer Forage	25	10	2.1	Searight's Dairy Farm	Monthly Quarterly	Monthly Quarterly Composite		Gamma-scan Sr-90
11. Soil	13	11	1.6	Meyer's Farm	Every 3 years (1982, 1985, etc.)	12 Core Samples 3" Deep (3" Dia. at each location (approx. 10' radius)		Gamma-scan Sr-90 Gross Beta Gross Alpha Uranium Isotopic
	30	4	0.6	Shippingport, Pa.				
	46	3	2.6	Industry, Pa.				
	32	15	0.8	(North of Site) Midland				
	48(a)	10	16.5	Weirton, W. Va.				
	51	5	8.0	Aliquippa, Pa.				
	47	14	4.8	E. Liverpool, Oh.				
	27	7	6.2	Brunton's Dairy				
	22	8	0.3	South of BVPS Site				
	29A	3	8.3	Nichol's Dairy				

* Additional dairies required by Environmental Technical Specification 3.2.1.D.2. In addition to Searight's (Site 25), three dairies are selected when milk is available based on highest deposition factors. Sites 27 and 29A are required for the Shippingport program.

TABLE V.A.1
CONSOLIDATED RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM
(Continued)Notes:

- (a) Control sample station: These are locations which are presumed to be outside the influence of plant effluents.
- (b) Typical LLD's for Gamma Spectrometry are shown in Table V.A.4.
- (c) In these cases a gamma isotopic analysis is done if the gross beta activity exceeds the reporting level of 0.53pCi/m^3 .
- (d) Analysis composites are well mixed actual samples prepared of equal portions from each shorter term samples from each location.
- (e) Composite samples are collected at intervals not exceeding 2 hours.
- (f) Weekly milk sample from Searight's Dairy is analyzed for I-131 only.
- (g) Milk samples are collected bi-weekly when animals are in pasture and monthly at other times. [Assume April - October for grazing season (pasture).]
- (h) The milk samples from Brunton's and Nicol's are collected once per month.
- (i) The fish samples will contain whatever species are available. If the available sample size permits, then the sample will be separated according to species and compositing will provide one sample of each species. If the available size is too small to make separation by species practical, then edible parts of all fish in the sample will be mixed to give one sample.
- (j) Composite samples are collected at intervals not exceeding 2 hours at locations 49.1 and 2.1. Weekly grab samples are obtained at location 3, 49 and 2A. A weekly grab sample is also obtained from daily composited grab samples obtained by the water treatment plant operator at location 5.
- (k) Two (2) TLD's are collected quarterly and annually from each monitoring location.

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Additional Notes:

- Sample points correspond to site numbers shown on maps.
- All Iodine I-131 analyses are performed within 40 hours of sample collection if possible.
- All Air samples are decayed for 72 hours before analyzing for Gross Beta.

ENVIRONMENTAL RADIOLOGICAL MONITORING PROGRAM SUMMARY

Name of Facility Duquesne Light Company Docket No. 50-334

Location of Facility Beaver, Pennsylvania Reporting Period Annual 1983
(County, State)

Medium or Pathway Sampled (Unit of Measurement)	Analysis and Total Number of Analysis Performed	Lower Limit of Detection (LLD)	All Indicator Locations		Location with Highest Quar. Mean Name Distance and Directions	Control Locations		Number of Nonroutine Reported Measurements***
			** Mean (f)	**Range		**Mean (f)	**Range	
Air Particulate and Radiiodine (X10 ⁻³ pCi/Cu. M.)	Gross (520) Beta	2.5	18(520/520) (6.1-44)		Sherman Dairy No. 28	19(52/52) (8.3-44)	18(52/52) (6.9-38)	0
	Sr-89 (40)	2	LLD		--	--	--	-
	Sr-90 (40)	0.1	0.13(3/40) (0.12-0.15)		Sherman Dairy No. 28	0.15(1/12) --	0.12(1/12) --	0
	I-131 (520)	10	LLD		--	--	--	-
	Gamma (120) Be-7	20	96(120/120) (31-151)		Sherman Dairy No. 28	104(12/12) (67-151)	98(12/12) (49-147)	0
	K-40	10	31(23/120) (12-87)		Industry, Pa No. 46 2.0 miles - NNE	70(2/12) (54-87)	23(1/12) --	0
	Ra-226	10	13(1/120)		Beaver Co. H. No. 29B 8 miles--ENE	13(1/12)	LLD	0
	Others	Table V.A. 4	LLD		--	--	--	-

* Nominal Lower Limit of Detection (LLD)

** Mean and range based upon detectable measurements only. Fraction of detectable measurements at specified locations is indicated in parentheses (f)

*** Nonroutine reported measurements are defined in Regulatory Guide 4.8 (December 1975) and the Beaver Valley Power Station Technical Specifications (Appendix B)

ENVIRONMENTAL RADIOLOGICAL MONITORING PROGRAM SUMMARY

Name of Facility Duquesne Light Company Docket No. 50-334

Location of Facility Beaver, Pennsylvania Reporting Period Annual 1983
(County, State)

Medium or Pathway Sampled (Unit of Measurement)	Analysis and Total Number of Analysis Performed	Lower Limit of Detection (LLD)	All Indicator Locations		Location with Highest Name Distance and Directions	Quar. Mean	Control Locations		Number of Nonroutine Reported Measurements***
			** Mean (f)	**Range		**Mean (f)	**Mean (f)	**Range	
							Montgomery Dam No. 49		
Sediment (pCi/g) (dry weight)	Gross (8)	0.3	17(8/8)		Montgomery Dam No. 49	22(2/2)	Same as		0
	Alpha		(9.7-34)		River Mile -- 31.0	(9.7-34)	High Location		
	Gross (8)	1.0	30(8/8)		SAPS Discharge No. 03	33(2/2)	27(2/2)		0
	Beta		(23-38)		River Mile -- 34.8	(29-38)	(23-31)		
	Sr-89 (8)	0.2	LLD		--	--	--		-
	Sr-90 (8)	0.04	0.076(5/8)		Montgomery Dam No. 49	0.099(1/2)	Same as		0
			(0.041-0.11)		River Mile -- 31.0	--	High Location		
	Gamma (8)								
	Be-7	0.2	0.51(3/8)		BVPS Discharge No. 02	0.69(1/2)	0.32(1/2)		0
			(0.32-0.69)		River Mile -- 35.0	--	--		
	K-40	0.5	12(8/8)		SAPS Discharge No. 03	13(2/2)	12(2/2)		0
			(8.9-16)		River Mile -- 34.8	(11-16)	(11-14)		
	Co-60	0.03	0.44(4/8)		BVPS Discharge No. 02	0.65(2/2)	LLD		0
			(0.22-0.88)		River Mile -- 35.0	(0.42-0.88)			
	Cs-137	0.02	0.33(8/8)		SAPS Discharge No. 03	0.40(2/2)	0.28(2/2)		0
			(0.19-0.50)		River Mile -- 34.8	(0.31-0.50)	(0.26-0.29)		
	Ra-226	0.1	2.3(8/8)		BVPS Discharge No. 02	3.0(2/2)	2.2(2/2)		0
			(1.4-4.1)		River Mile -- 35.0	(1.9-4.1)	(2.1-2.3)		
	Th-232	0.02	1.3(8/8)		BVPS Discharge No. 02	1.7(2/2)	1.2(2/2)		0
			(1.0-1.8)		River Mile -- 35.0	(1.6-1.8)	(1.1-1.2)		

* Nominal Lower Limit of Detection (LLD)

** Mean and range based upon detectable measurements only. Fraction of detectable measurements at specified locations is indicated in parentheses (f)

*** Nonroutine reported measurements are defined in Regulatory Guide 4.8 (December 1975) and the Beaver Valley Power Station Technical Specifications (Appendix B)

ENVIRONMENTAL RADIOLOGICAL MONITORING PROGRAM SUMMARY

Name of Facility Duquesne Light Company Docket No. 50-334

Location of Facility Beaver, Pennsylvania Reporting Period Annual 1983
(County, State)

Medium or Pathway Sampled (Unit of Measurement)	Analysis and Total Number of Analysis Performed	Lower Limit of Detection (LLD)	All Indicator Locations		Location with Highest Quar. Mean		Control Locations		Number of Nonroutine Reported Measurements***
			** Mean (f)	**Range	Name	**Mean (f)	**Mean (f)	**Range	
					Distance and Directions	**Range			
							Montgomery Dam No. 49		
Sediment (pCi/g) (dry weight) (continued)	Others	Table V.A. 4	LLD	--	--	--	--	--	-
	U-233 (8) and U-234	0.01	0.63(8/8) (0.38-0.90)		BVPS Discharge No. 2A River Mile -- 34.8	0.71(2/2) (0.54-0.90)	0.51(2/2) (0.48-0.55)		0
	U-235 (8)	0.01	0.020(8/8) (0.011-0.040)		N. Cumberland Dam No. 50 River Mile -- 54.0	0.026(2/2) (0.011-0.040)	0.017(2/2) (0.013-0.020)		0
	U-238 (8)	0.01	0.31(8/8) (0.033-0.62)		SAPS Discharge No. 3 River Mile -- 34.8	0.50(2/2) (0.38-0.62)	0.37(2/2) (0.35-0.38)		0

* Nominal Lower Limit of Detection (LLD)

** Mean and range based upon detectable measurements only. Fraction of detectable measurements at specified locations is indicated in parentheses (f)

*** Nonroutine reported measurements are defined in Regulatory Guide 4.8 (December 1975) and the Beaver Valley Power Station Technical Specifications (Appendix B)

SECTION V - A

1983 Annual Radiological Environmental Report

TABLE V.A.2

1983 Annual Radiological Environmental Report

ENVIRONMENTAL RADIOLOGICAL MONITORING PROGRAM SUMMARY

Name of Facility Duquesne Light Company Docket No. 50-334
 Location of Facility Beaver, Pennsylvania Reporting Period Annual 1983
 (County, State)

Medium or Pathway Sampled (Unit of Measurement)	Analysis and Lower Limit Total Number of of Analysis Detection (LLD)	All Indicator Locations ** Mean (f) ** Range	Location with Highest Quar. Mean Name Distance and Direction Control Locations ** Mean (f) ** Range	Number of Nonroutine Reported Measurements***
Feed and Forage (pCi/g) (dry weight)	Sr-90 (4) 0.003	0.075(4/4) (0.020-0.20)	Searight Dairy No. 25 2.4 miles - SW One sample location	0
	Gamma (12) Be-7 0.3	0.76(6/12) (0.45-0.91)	-- --	0
	K-40 0.5	12(12/12) (4.3-23)	-- --	0
	Cs-137 0.03	0.045(5/12) (0.015-0.083)	-- --	0
	Ra-226 0.03	0.72(1/12) --	-- --	0
	Th-228 0.05	0.064(2/12) (0.037-0.096)	-- --	0
Others	Table V.A. 4	LLD	-- --	0

* Nominal Lower Limit of Detection (LLD)

** Mean and range based upon detectable measurements only. Fraction of detectable measurements at specified locations is indicated in parentheses (f)

*** Nonroutine reported measurements are defined in Regulatory Guide 4.8 (December 1975) and the Beaver Valley Power Station Technical Specifications (Appendix B)

ENVIRONMENTAL RADIOLOGICAL MONITORING PROGRAM SUMMARY

Name of Facility Duquesne Light Company Docket No. 50-334Location of Facility Beaver, Pennsylvania Reporting Period Annual 1983
(County, State)

Medium or Pathway Sampled (Unit of Measurement)	Analysis and Total Number of Analysis Performed	Lower Limit of Detection (LLD)	All Indicator Locations	Location with Highest Quar. Mean	Control Locations		Number of Nonroutine Reported Measurements***
			** Mean (f)	Name	**Mean (f)	**Mean (f)	
			**Range	Distance and Directions	**Range	**Range	
						Weirton WV No. 48	
Food and Garden Crops (pCi/g) (wet weight)	I-131 (8)	0.006	LLD	--	--	--	-
	Gamma (8)						
	Be-7	0.3	0.17(1/8) --	Shippingport PA No. 10	0.17(1/2) --	LLD	0
	K-40	0.5	4.6(8/8) (1.9-12)	Weirton WV No. 48 20 miles - SW	5.9(3/3) (3.4-12)	Same as High Location	0
	Cs-137	0.01	0.020(1/8) --	Georgetown PA No. 15 4.6 miles -- WNW	0.020(1/1) --	LLD	0
	Others	Table V.A. 4	LLD	--	--	--	0

* Nominal Lower Limit of Detection (LLD)

** Mean and range based upon detectable measurements only. Fraction of detectable measurements at specified locations is indicated in parentheses (f)

*** Nonroutine reported measurements are defined in Regulatory Guide 4.8 (SEPTEMBER 1975) and the Beaver Valley Power Station Technical Specifications (Appendix B)

ENVIRONMENTAL RADIOLOGICAL MONITORING PROGRAM SUMMARY

Name of Facility Duquesne Light Company Docket No. 50-334
Location of Facility Beaver, Pennsylvania Reporting Period Annual 1983
(County, State)

Medium or Pathway Sampled (Unit of Measurement)	Analysis and Total Number of Analysis Performed	Lower Limit of Detection (LLD)	All Indicator Locations	Location with Highest Quar.	Control Locations	Number of Nonroutine Reported Measurements***	
			** Mean (f)	Name	**Mean (f)		**Mean (f)
			**Range	Distance and Directions	**Range		**Range
					Brunton Dairy No. 27		
Milk (pCi/g)	I-131 (153)		LLD	--	--	--	
	Sr-89 (123)		LLD	--	--	--	
	Sr-90 (123)		4.6(123/123) (0.43-18.0)	Collins No. 69	10.1(19/19) (3.9-18.0)	0	
	Gamma (124) K-40		1340(124/124) (686-2060)	Foxall No. 98	1710(8/8) (1270-2060)	1330(12/12) (1070-1530)	0
	Cs-137		7.4(16/124) (4.0-13.6)	Nichols No. 29A	10.9(1/12) --	LLD	0
	Others	Table V.A. 4	LLD	--	--	--	--

* Nominal Lower Limit of Detection (LLD)

** Mean and range based upon detectable measurements only. Fraction of detectable measurements at specified locations is indicated in parentheses (f)

*** Nonroutine reported measurements are defined in Regulatory Guide 4.8 (December 1975) and the Beaver Valley Power Station Technical Specifications (Appendix B)

ENVIRONMENTAL RADIOLOGICAL MONITORING PROGRAM SUMMARY

Name of Facility Duquesne Light Company Docket No. 50-334
 Location of Facility Beaver, Pennsylvania Reporting Period Annual 1983
 (County, State)

Medium or Pathway Sampled (Unit of Measurement)	Analysis and Lower Limit		All Indicator locations	Location with Highest Quar. Mean	Control Locations	Number of Nonroutine Reported Measurements***	
	Total Number of Analysis Performed	Detection (LLD)	** Mean (f) **Range	Name Distance and Directions **Mean (f) **Range	**Mean (f) **Range		
					Weirton, WV No. 48		
External Radiation (mR/day)	(175) (quarterly)	0.05	0.18(175/175) (0.12-0.22)	No. 87	0.20(4/4) (0.19-0.21)	0.19(4/4) (0.18-0.20)	0
	(44 annual)	0.05	0.17(44/44) (0.13-0.20)	No. 88	0.20(1/1) --	0.18(1/1) --	0
					Montgomery Dam No. 49		
Fish (pCi/g) (wet weight)	Gamma (B)						
	K-40	0.5	2.1(8/8) (0.43-3.2)	Montgomery Dam No. 49 River Mile -- 31.0	2.2(3/3) (0.62-3.0)	Same as High Location	0 -
	Cs-137	0.01	0.039(1/8) --	BVPS Discharge No. 02A River Mile -- 35.0	0.039(1/2) --	LLD	0 -
	Others	Table V.A.4	LLD	--	--	--	-

* Nominal Lower Limit of Detection (LLD)

** Mean and range based upon detectable measurements only. Fraction of detectable measurements at specified locations is indicated in parentheses (f)

*** Nonroutine reported measurements are defined in Regulatory Guide 4.8 (December 1975) and the Beaver Valley Power Station Technical Specifications (Appendix B)

ENVIRONMENTAL RADIOLOGICAL MONITORING PROGRAM SUMMARY

Name of Facility Duquesne Light Company Docket No. 50-334
 Location of Facility Beaver, Pennsylvania Reporting Period Annual 1983
 (County, State)

Medium or Pathway Sampled (Unit of Measurement)	Analysis and Total Number of Analysis Performed	Lower Limit of Detection (LLD)	All Indicator Locations	Location with Highest Quar. Mean	Control Locations		Number of Nonroutine Reported Measurements***
			** Mean (f)	Name	**Mean (f)	**Mean (f)	
			**Range	Distance and Directions	**Range	**Range	
Montgomery Dam No. 49							
Surface Water (pCi/l)	Gross (72) Alpha	2	9.1(1/72) --	Arco Polymers No. 49.1	9.1(1/12) --	LLD	-
	Gross (72) Beta	1	5.3(71/72) (1.2-15)	BVPS Discharge No. 02A River Mile -- 35.0	6.7(12/12) (3.9-11)	5.2(12/12) (3.6-6.9)	0
	Gamma (72)	Table V.A.4	LLD	--	--	--	-
	Sr-89 (24)	2	LLD	--	--	--	-
	Sr-90 (24)	0.5	LLD	--	--	--	-
	Co-60 (24)(a)	1	LLD	--	--	--	-
	Tritium (24)	100	2900(20/24) (120-24100)	BVPS Discharge No. 02A River Mile -- 35.0	11100(4/4) (920-24100)	190(4/4) (130-290)	0 -

(a) Co-60 analyzed by high sensitivity method.

* Nominal lower Limit of Detection (LLD)

** Mean and range based upon detectable measurements only. Fraction of detectable measurements at specified locations is indicated in parentheses (f)

*** Nonroutine reported measurements are defined in Regulatory Guide 4.8 (December 1975) and the Beaver Valley Power Station Technical Specifications (Appendix B)

ENVIRONMENTAL RADIOLOGICAL MONITORING PROGRAM SUMMARY

Name of Facility Duquesne Light Company Bucket No. 50-314
 Location of Facility Beaver, Pennsylvania Reporting Period Annual 1983
 (County, State)

Medium or Pathway Sampled (Unit of Measurement)	Analysis and Total Number of Analysis Performed	Lower Limit of Detection (LLD)	All Indicator Locations ** Mean (f) ** Range	Location with Highest Name Distance and Direction	Quar. Mean ** Mean (f) ** Range	Control Locations ** Mean (f) ** Range	Number of Nonroutine Reported Measurements***
Drinking Water (pCi/L)	I-131 (103)	0.2	0.34(3/103) (0.23-0.48)	Midland Pa. No. 04 River Mile -- 30.3	0.34(3/51) (0.23-0.48)	--	0
	Gross (24) Alpha	0.6	LLD	--	--	--	--
	Gross (24) Beta	1	4.1(24/24) (1.4-9.0)	Midland Pa. No. 4 River Mile -- 30.3	4.5(12/12) (1.7-9.0)	--	0
	Gamma (103)	Table V.A. 4	LLD	--	--	--	--
	Sr-89 (8)	1.5	LLD	--	--	--	--
	Sr-90 (8)	0.5	LLD	--	--	--	--
	Co-60 (8)(a)	1	LLD	--	--	--	--
	Tritium (8)	100	200(6/8) (100-350)	Midland Pa. No. 04 River Mile -- 30.3	220(2/4) (170-270)	--	0

(a) Co-60 analyzed by high sensitivity method.

* Nominal Lower Limit of Detection (LLD)

** Mean and range based upon detectable measurements only. Fraction of detectable measurements at specified locations is indicated in parentheses (f)

*** Nonroutine reported measurements are defined in Regulatory Guide 4.8 (December 1975) and the Beaver Valley Power Station Technical Specifications (Appendix B)

ENVIRONMENTAL RADIOLOGICAL MONITORING PROGRAM SUMMARY

Name of Facility Duquesne Light Company Docket No. 50-334Location of Facility Beaver, Pennsylvania Reporting Period Annual 1983
(County, State)

Medium or Pathway Sampled (Unit of Measurement)	Analysis and Total Number of Analysis Performed	Lower Limit of Detection (LLD)	All Indicator Locations	Location with Highest Quar. Mean		Control Locations		Number of Nonroutine Reported Measurements***
			** Mean (f)	Name	**Mean (f)	**Mean (f)		
			**Range	Distance and Directions	**Range	**Range		
Georgetown, Pa. No. 15								
Ground Water (pCi/liter)	Gross (16) Alpha	2	LLD	--	--	--	--	-
	Gross (16) Beta	1	3.0(16/16) (1.0-5.8)	Hookstown Pa. No. 14 3 miles -- WSW	4.3(4/4) (3.4-5.8)	2.4(4/4) (1.4-3.6)	--	0
	Gamma (16)	Table V.A.4	LLD	--	--	--	--	-
	Tritium (16)	90	170(13/16) (90-260)	Shippingport Pa No. 11 0.8 mile -- NE	190(4/4) (110-260)	140(4/4) (90-190)	--	0

* Nominal Lower Limit of Detection (LLD)

** Mean and range based upon detectable measurements only. Fraction of detectable measurements at specified locations is indicated in parentheses (f)

*** Nonroutine reported measurements are defined in Regulatory Guide 4.8 (December 1975) and the Beaver Valley Power Station Technical Specifications (Appendix B)

V. ENVIRONMENTAL MONITORINGA. Environmental Radioactivity Monitoring Program (continued)2. Summary of Results

All results of this monitoring program are summarized in Table V.A.2. This table is prepared in the format specified by NRC Regulatory Guide 4.8 and in accordance with Beaver Valley Power Station Operating License, (Appendix B, Environmental Technical Specifications). Summaries of results of analysis of each media are discussed in Sections V-B through V-H and an assessment of radiation doses are found in Section V-I. Table V.A.3 summarizes Beaver Valley Power Station pre-operational ranges for the various sampling media during the years 1974 and 1975. Comparisons of pre-operational data with operational data indicate the ranges of values are in good agreement for both periods of time.

In a few cases where activity was detected, some of the activity was attributable to previous nuclear weapons tests and the remaining detected activities were near the lower limit of their detection (LLD) and are attributable to the normal statistical fluctuation near the LLD level.

The conclusion from all program data is that the operation of the Shippingport and Beaver Valley Power Station has not resulted in any detectable changes to the environment attributable to either station.

3. Quality Control Program

The Quality Control Program implemented by Duquesne Light Company to assure reliable performance by the DLC contractor and the supporting QC data are presented and discussed in Section III of this report. The lower limits of detection for various analysis for each media monitored by this program by the DLC Contractor Laboratory are provided in Table V.A.4.

TABLE V.A.3 (Page 1 of 4)

ENVIRONMENTAL RADIOLOGICAL MONITORING PROGRAM SUMMARY

Name of Facility Shippensburg Atomic Power Station Docket No. Not ApplicableName of Facility Beaver Valley Power Station Docket No. 50-111Location of Facility Beaver, Pennsylvania Reporting Period OT 1974 - 1975
(County, State)PRE-OPERATIONAL PROGRAM SUMMARY (COMBINED 1974 - 1975)

Medium or Pathway Sampled (Unit of Measurement)	Analysis and Total Number of Analysis Performed		Lower Limit of Detection LLD	All Indicator Locations Mean, (f) Range		
Surface Water pCi/l	Gross Alpha	(40)	0.3	0.75	5/40	0.6 - 1.1
	Gross Beta	(120)	0.6	4.4	120/120	2.5 - 11.4
	Gamma	(1)	10 - 60		< LLD	
	Tritium	(121)	100	300	120/121	130 - 300
	Sr-89	(0)	—		—	
	Sr-90	(0)	—		—	
	C-14	(0)	—		—	
Drinking Water pCi/l	I-131	(0)	—			
	Gross Alpha	(50)	0.3	0.6	4/50	0.4 - 0.8
	Gross Beta	(208)	0.6	1.8	208/208	2.3 - 6.4
	Gamma	(0)	—		—	
	Tritium	(211)	100	310	211/211	130 - 1000
	C-14	(0)	—		—	
	Sr-89	(0)	—		—	
Ground Water pCi/l	Gross Alpha	(19)	0.3		< LLD	
	Gross Beta	(76)	0.6	1.9	73/75(a)	1.3 - 3.6
	Tritium	(81)	100	440	77/81	80 - 300
	Gamma	(1)	10 - 60		< LLD	
Air Particulates and Gaseous pCi/m ³	Gross Alpha	(188)	0.001	0.003	35/188	0.002 - 0.004
	Gross Beta	(927)	0.006	0.07	927/927	0.02 - 0.07
	Sr-89	(0)			—	
	Sr-90	(0)			—	
	I-131	(816)	0.04	0.08	2/816	0.07 - 0.08
	Gamma	(197)				
	TcNb-95		0.005	0.04	122/197	0.01 - 0.16
	Ru-106		0.010	0.04	50/197	0.02 - 0.07
	Ce-141		0.010	0.02	3/197	0.01 - 0.02
	Ce-144		0.010	0.02	44/197	0.01 - 0.04
	Others				< LLD	

TABLE V.A.3 (Page 2 of 4)
ENVIRONMENTAL RADIOLOGICAL MONITORING PROGRAM SUMMARY

Name of Facility Shippingport Atomic Power Station Docket No. Not Applicable
 Name of Facility Beaver Valley Power Station Docket No. 50-134
 Location of Facility Beaver, Pennsylvania Reporting Period CT 1974 - 1975
 (County, State)

PRE-OPERATIONAL PROGRAM SUMMARY (COMBINED 1974 - 1975)

Medium or Pathway Sampled (Unit of Measurement)	Analysis and Total Number of Analysis Performed		Lower Limit of Detection LLD	All Indicator Locations Mean, (f) Range	
Soil pCi/g (dry) (Template Samples)	Gross Alpha	(0)	—	—	
	Gross Beta	(64)	1	22	64/64 14 - 32
	Sr-89	(64)	0.25	0.4	1/64 —
	Sr-90	(64)	0.05	0.3	48/64 0.1 - 1.3
	U-234,235,238	(0)	—	—	
	Gamma	(64)			
	K-40		1.5	13	63/64 5 - 24
	Cs-137		0.1	1.5	56/64 0.1 - 6.8
	Cs-144		0.3	1.1	7/64 0.2 - 3
	ZrNb-95		0.05	0.3	13/64 0.1 - 2
	Ru-106 ^(b)		0.3	1.1	3/64 0.5 - 2
	Others				< LLD
Soil pCi/g (dry) (Core Samples)	Gross Alpha	(0)	—	—	
	Gross Beta	(8)	1	21	8/8 16 - 28
	Sr-89	(8)	0.25		< LLD
	Sr-90	(8)	0.05	0.2	5/8 0.08 -
	Gamma	(8)			
	K-40		1.5	13	3/8 7 - 20
	Cs-137		0.1	1.2	7/8 0.2 - 2
	Co-60		0.1	0.2	1/8 —
	Others				< LLD

TABLE V.A.3 (Page 3 of 4)
ENVIRONMENTAL RADIOLOGICAL MONITORING PROGRAM SUMMARY

Name of Facility Shippingport Atomic Power Station Docket No. Not Applicable
 Name of Facility Beaver Valley Power Station Docket No. 50-133
 Location of Facility Beaver, Pennsylvania Reporting Level CY 1974 - 1975
 (County, State)

PRE-OPERATIONAL PROGRAM SUMMARY (COMBINED 1974 - 1975)

Medium or Pathway Sampled (Unit of Measurement)	Analysis and Total Number of Analysis Performed	Lower Limit of Detection LLD	All Indicator Locations Mean, (f) Range		
Sediments pCi/g (dry)	Gross Alpha (0)	—			
	Gross Beta (33)	1	18	33/33	5 - 30
	Sr-90 (0)	—		—	
	U-234, 235, 238 (0)	—		—	
	Gamma (33)		13	33/33	2 - 30
	K-40	1.5	13	33/33	2 - 30
	Cs-137	0.1	0.4	21/33	0.1 - 0.6
	ZrNb-95	0.05	0.8	12/33	0.2 - 3.2
	Ce-144	0.3	0.5	3/33	0.4 - 0.7
	Ru-106 ^(b)	0.3	1.5	3/33	1.3 - 1.3
	Others				< LLD
Foodstuff pCi/g (dry)	Gamma (8)				
	K-40	1	33	8/8	10 - 53
	Cs-137	0.1	0.2	1/8	—
	ZrNb-95	0.05	0.2	1/8	—
	Ru-106 ^(b)	0.3	0.3	1/8	—
	Others				< LLD
Feedstuff pCi/g (dry)	Gross Beta (80)	0.05	19	80/80	8 - 50
	Sr-89 (81)	0.025	0.2	33/81	0.04 - 0.93
	Sr-90 (81)	0.005	0.4	78/81	0.02 - 0.81
	Gamma (81)				
	K-40	1	19	75/81	5 - 46
	Cs-137	0.1	0.5	6/81	0.2 - 1.6
	Ce-144	0.3	1.5	5/81	0.9 - 2.6
	ZrNb-95	0.05	0.3	13/81	0.2 - 1.3
	Ru-106 ^(b)	0.3	1.4	12/81	0.6 - 1.3
	Others				< LLD

TABLE V.A.3 (Page 4 of 4)
ENVIRONMENTAL RADIOLOGICAL MONITORING PROGRAM SUMMARY

Name of Facility Shippingport Atomic Power Station Docket No. Not Applicable
 Name of Facility Beaver Valley Power Station Docket No. 50-334
 Location of Facility Beaver, Pennsylvania Reporting Level CY 1974 - 1975
 (County, State)

PRE-OPERATIONAL PROGRAM SUMMARY (COMBINED 1974 - 1975)

Medium or Pathway Sampled (Unit of Measurement)	Analysis and Total Number of Analysis Performed		Lower Limit of Detection LLD	All Indicator Locations Mean, (#) Range		
Milk pCi/l	I-131	(91)	0.25	0.6	⁴ /91	0.3 - 0.8
	Sr-89	(134)	5	7	⁴ /134	6 - 11
	Sr-90	(134)	1	5.3	¹³² /134	1.5 - 12.8
	Gamma	(134)				
	Cs-137		10	13	¹⁹ /134	11 - 16
	Others					< LLD
External Radiation mR/day	γ - Monthly	(599)	0.5 mR *	0.20	⁵⁹⁹ /599	0.08 - 0.51
	γ - Quarterly	(195)	0.5 mR *	0.20	¹⁹⁵ /195	0.11 - 0.38
	γ - Annual	(48)	0.5 mR *	0.19	⁴⁸ /48	0.11 - 0.30
Fish pCi/g (wet)	Gross Beta	(17)	0.01	1.9	¹⁵ /17	1.0 - 3.2
	Sr-90	(17)	0.005	0.14	¹⁷ /17	0.02 - 0.10
	Gamma	(17)				
	K-40		0.5	2.4	¹⁷ /17	1.0 - 3.2
	Other					< LLD

* LLD in units of MR - Lower end of useful integrated exposure detectability range for a passive radiation detector (TLD).

- (a) One outlier not included in mean. (Water taken from dried-up spring with high sediment and potassium content. Not considered typical groundwater sample.)
 (b) May include Ru-106, Ru-103, Be-7.

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TABLE V.A.4

TYPICAL LLDs * FOR GAMMA SPECTROMETRY

Nuclide	Milk Water (pCi/liter)	Air Particulates (10 ⁻³ pCi/m)	Vegetation (pCi/kg dry)	Sediment & Soil (pCi/g dry)	Fish (pCi/g wet)
Be-7	30	20	50	0.03	0.05
K-40	60	20	**	**	**
Cr-51	40	10	100	0.05	0.1
Mn-54	3	0.5	30	0.02	0.03
Co-58	3	0.6	30	0.02	0.03
Fe-59	6	1	60	0.03	0.06
Co-60	3	0.6	30	0.02	0.03
Zn-65	8	1	70	0.04	0.07
Zr/Nb-95	5	2	50	0.03	0.05
Ru-103	3	2	40	0.03	0.04
Ru-106	30	5	30	0.02	0.03
Ag-110m	5	3	30	0.02	0.03
I-131	4	2	30	0.02	0.03
Te-132	1	2	20	0.01	0.02
I-135	4	2	20	0.01	0.02
Ca-134	4	0.6	30	0.02	0.03
Cs-136	6	0.6	50	0.03	0.05
Cs-137	4	0.6	20	0.02	0.03
Ba/La-140	10	6	40	0.02	0.04
Ce-141	6	2	60	0.03	0.06
Ce-144	30	5	200	0.1	0.2
Ba-226	60	6	600	0.3	0.6
Th-228	10	1	60	0.03	0.06

* At time of analysis (DLC Contractor Lab).

** Activity detected in all samples.

NOTE: Lower level of Detection is defined in Beaver Valley Power Station Technical Specifications.

V. ENVIRONMENTAL MONITORINGB. Air Monitoring1. Characterization of Air and Meteorology

The air in the vicinity of the site contains pollutants typical for an industrial area. Air flow is generally from the Southwest in summer and from the Northwest in the winter.

2. Air Sampling Program and Analytical Techniquesa. Program

The air is sampled for gaseous radioiodine and radioactive particulates at each of ten (10) off-site air sampling stations. The locations of these stations are listed in Table V.A.1 and shown on a map in Figure 5.B.1.

Samples are collected at each of these stations by continuously drawing about one cubic foot per minute of atmosphere air through a glass fiber filter and through a charcoal cartridge. The former collects airborne particulates; the latter is for radioiodine sampling. Samples are collected for analysis on a weekly basis.

The charcoal is used in the weekly analysis of airborne I-131. The filters are analyzed each week for gross beta, then composited by station for monthly analysis by gamma spectrometry. They are further composited in a quarterly sample from each station for Sr-89/90 analysis. In order to reduce interference from natural radon and thoron radioactivities, all filters are allowed to decay for a few days after collection prior to counting for beta in a low background counting system.

b. Procedures

Gross Beta analysis is performed by placing the filter paper from the weekly air sample in a 2" x 1/4" planchet and counting it in a low background, gas flow proportional counter.

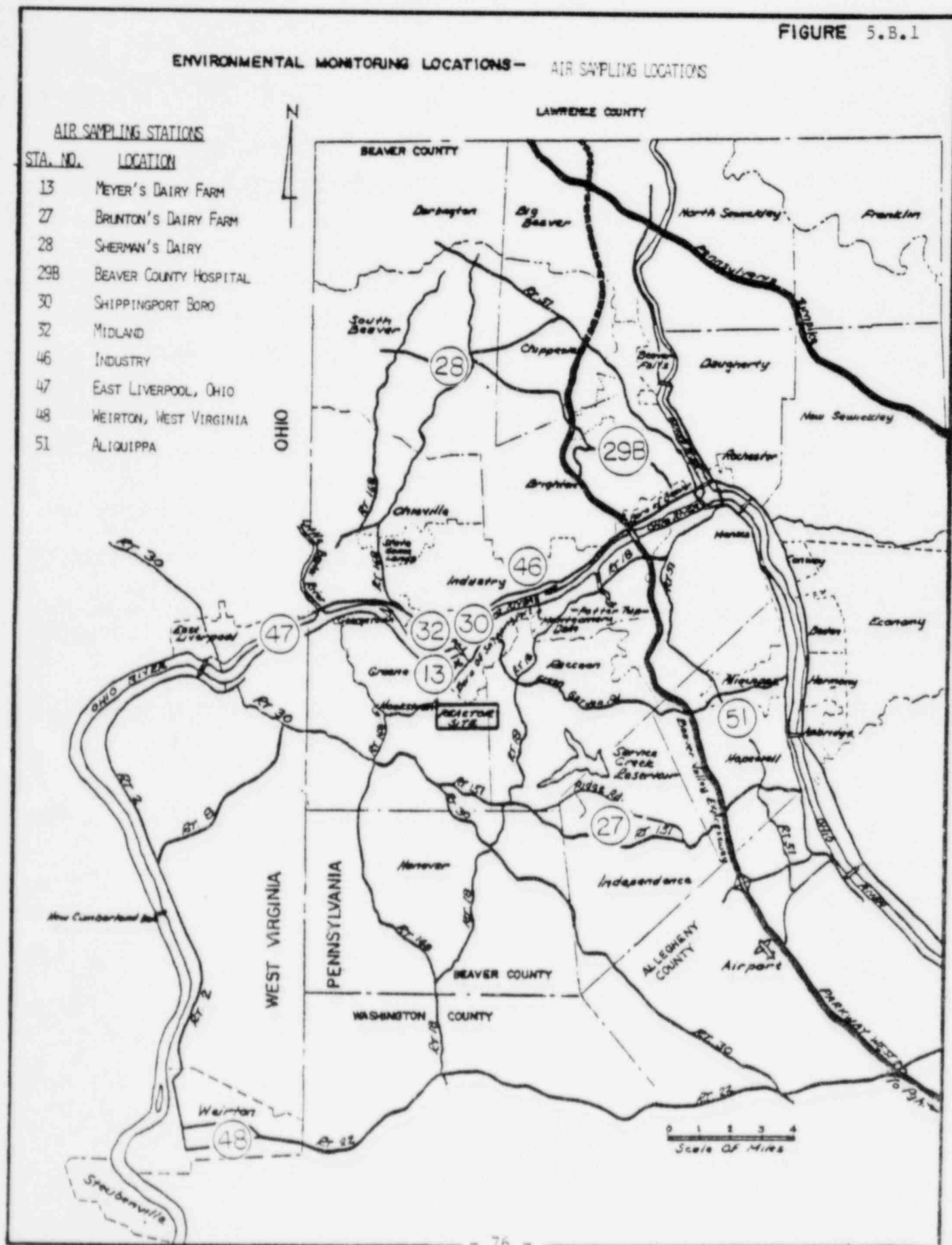
b. Procedures (continued)

Gamma emitters are determined by stacking all the filter papers from each monitoring station collected during the month and scanning this composite on a lithium drifted germanium (Ge(Li)) gamma spectrometer.

Radioiodine (I-131) analysis is performed by a gamma scan of the charcoal in a weekly charcoal cartridge. The activity is referenced to the mid-collection time.

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FIGURE 5.B.1



V. ENVIRONMENTAL MONITORINGb. Procedures (continued)

Strontium-89 and Strontium-90 activities are determined in quarterly composited air particulate filters. Stable strontium carrier is added to the sample and it is leached in nitric acid to bring deposits into solution. The mixture is then filtered. Half of the filtrate is taken for strontium analysis and is reduced in volume by evaporation. Strontium is precipitated as $\text{Sr}(\text{NO}_3)_2$ using fuming (90%) nitric acid. An iron (ferric hydroxide) scavenge is performed, followed by addition of stable yttrium carrier and a 5 to 7 day period for yttrium ingrowth. Yttrium is then precipitated as hydroxide, is dissolved and re-precipitated as oxalate. The yttrium oxalate is mounted on a nylon planchet and is counted in a low level beta counter to infer strontium-90 activity. Strontium-89 activity is determined by precipitating SrCO_3 from the sample after yttrium separation. This precipitate is mounted on a nylon planchet and is covered with 80 mg/cm² aluminum absorber for level beta counting.

3. Results and Conclusions

A summary of data is presented in Table V.A.2.

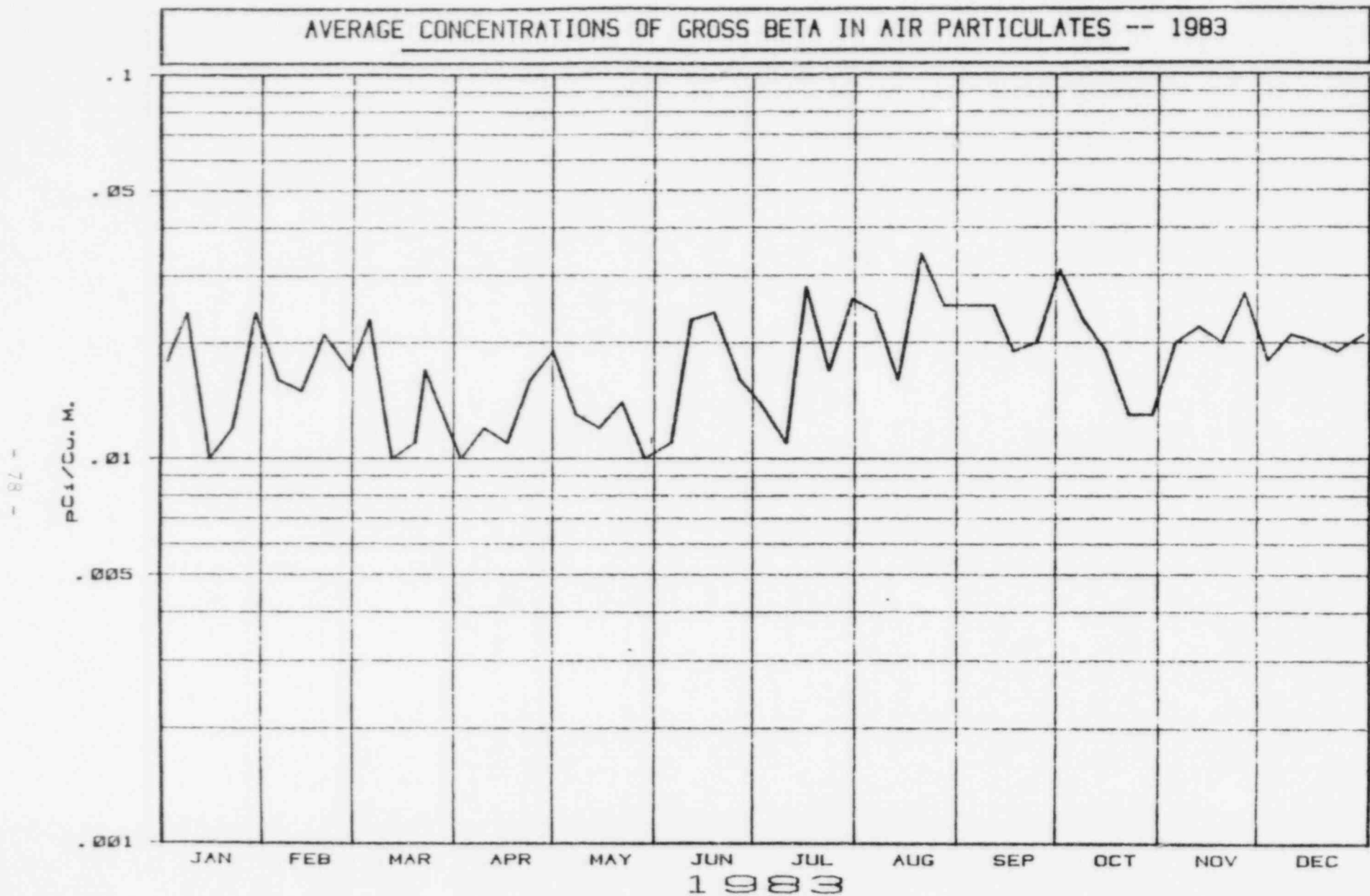
a. Airborne Radioactive Particulates

A total of five hundred twenty (520) weekly samples from ten (10) locations were analyzed for gross beta. Results were comparable to previous years. Figure 5.B.2 illustrates the average concentration of gross beta in air particulates.

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FIGURE 5.B.2



V. ENVIRONMENTAL MONITORING

a. Airborne Radioactive Particulates (continued)

The weekly air particulate samples were composited to one hundred and twenty (120) monthly samples which were analyzed by gamma spectrometry. Naturally occurring Be-7 was present in every sample. Occasional traces above detection levels of other nuclides were present. Some were natural, others were residual from previous nuclear weapons tests. These are listed in the summary Table V.A.2. Examination of effluent data from the Beaver Valley Power Station and the Shippingport Atomic Power Station demonstrated that none of the slightly elevated results are attributable to the operation of either power station.

A total of forty (40) quarterly samples were each analyzed for Sr-89, and Sr-90.

Based on the analytical results, the operation of Beaver Valley Power Station and Shippingport Atomic Power Station did not contribute to any increase in air particulate radioactivity during CY 1983.

b. Radioiodine

A total of five hundred and twenty (520) weekly charcoal filter samples were analyzed for I-131. No detectable concentrations were found at any locations.

Based on analytical results, the operation of Beaver Valley Power Station and Shippingport Atomic Power Station did not contribute to any increase in airborne radioiodine during CY 1983.

V. ENVIRONMENTAL MONITORINGC. Monitoring of Sediments and Soils (Soil Monitoring is required every 3 years and was not required in 1983.)1. Characterization of Stream Sediments

The stream sediments consist largely of sand and silt. Soil samples may vary from sand and silt to a heavy clay with variable amounts of organic material.

2. Sampling Program and Analytical Techniques

a. Program

River bottom sediments were collected semi-annually above the Montgomery Dam in the vicinities of the Beaver Valley discharge and Shippingport discharge and above the New Cumberland Dam. A Ponar or Eckman dredge is used to collect the sample. The sampling locations are also listed in Table V.A.1 and are shown in Figure 5.C.1.

Bottom sediments are analyzed for gross alpha and beta activity, strontium, uranium, and the gamma-emitting radionuclides.

b. Analytical Procedures

Gross beta - sediments and soils are analyzed for gross beta by mounting a 1 gram portion of dried sediment in a 2" planchet. The sample is counted in a low background, gas flow proportional counter. Self absorption corrections are made on the basis of sample weight.

Gross alpha activity of sediment or soil is analyzed in the same manner as gross beta except that the counter is set up to count only alpha.

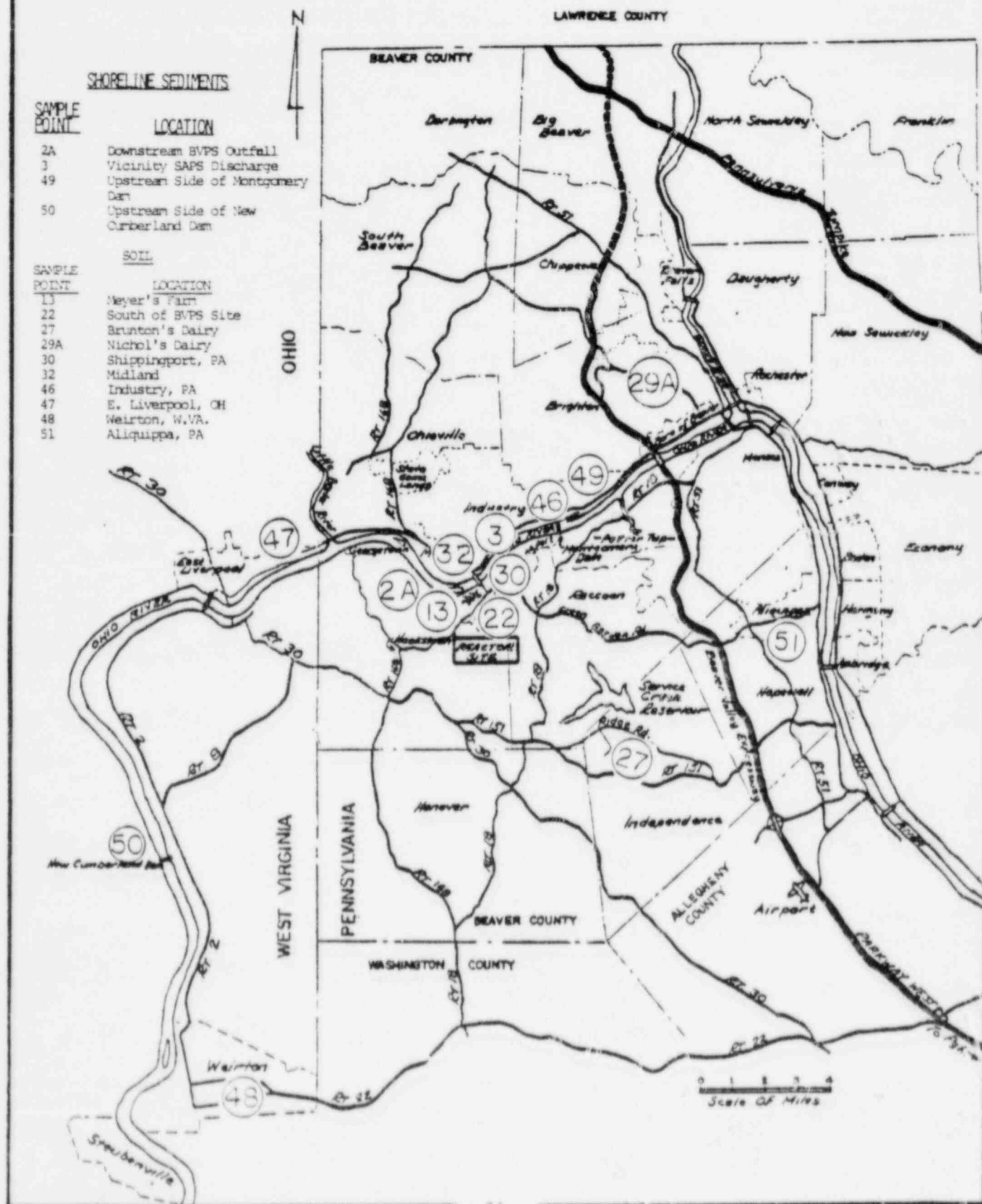
Gamma analysis of sediment or soil is performed in a 100 ml plastic bottle which is counted by a gamma spectrometer.

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SEDIMENTS AND SOILS

FIGURE 5.C.1

ENVIRONMENTAL MONITORING LOCATIONS— SHORELINE SEDIMENTS & SOIL



V. ENVIRONMENTAL MONITORING

b. Analytical Procedures (continued)

Strontium 89 and 90 are determined by radiochemistry. A weighed sample of sediment or soil is leached with Nitric Acid HNO_3 . A stable carrier is added for determination of recovery. Strontium concentration and purification is ultimately realized by precipitations of strontium nitrate in fuming nitric acid. Additional hydroxide precipitations and barium chromate separations are also used. The purified strontium is converted to a carbonate for weighing and counting. Samples are counted soon after separation (5 - 7 days is allowed for yttrium ingrowth). Activities are calculated on the basis of appropriate Sr-89 decay and Y-90. Separate mounts covered with a 80 mg/cm aluminum absorber are used for counting in a low background beta counter.

Uranium isotopic analysis of sediment and soil samples were performed by alpha spectrometry after leaching and isolation of the uranium by an ion exchange chromatography plus mercury cathode electrolysis, then electroplated onto a planchet.

3. Results and Conclusions

a. Results

The results of sediment analysis are summarized in Table V.A.2.

There were no significant differences between these current levels and those previously detected in both upstream and downstream sediment samples.

V. ENVIRONMENTAL MONITORING3. Results and Conclusions (continued)

a. Results (continued)

Uranium isotopic analyses were performed by alpha spectroscopy. The results suggest that only naturally occurring U-234 and U-238 were present since the activities were nearly always the same in each sample and the levels are within the expected range of natural uranium activities. In equilibrium, U-234 and U-238 have the same activity.

b. Conclusion

Other than trace amounts of Co-60 which have been detected since 1977, the sediment analyses do not indicate any radioactivity attributable to Beaver Valley Power Station and Shippingport Atomic Power Station liquid discharge. Small amounts of Cs-137 from weapons testing fallout was found in all river sediment samples including those upstream above Montgomery Dam which are unaffected by plant effluents.

V. ENVIRONMENTAL MONITORINGD. Monitoring of Feedcrops and Foodcrops1. Characterization of Vegetation and Foodcrops

According to a survey made in 1981, there were approximately 650 farms in Beaver County. The principle source of revenue for the farms was in dairy products which amounted to nearly \$4,659,000.00. Revenues from other farm products were as follows:

Crops	\$2,143,000.00
Horticulture	\$ 551,000.00
Meat	\$1,274,000.00
Poultry	\$ 392,000.00

The percentage of crop land in Beaver County is approximately 17%, pasture land - 6.5%, forest land - 47.8%, and other land uses - 28.7%.

2. Sampling Program and Analytical Techniques

a. Program

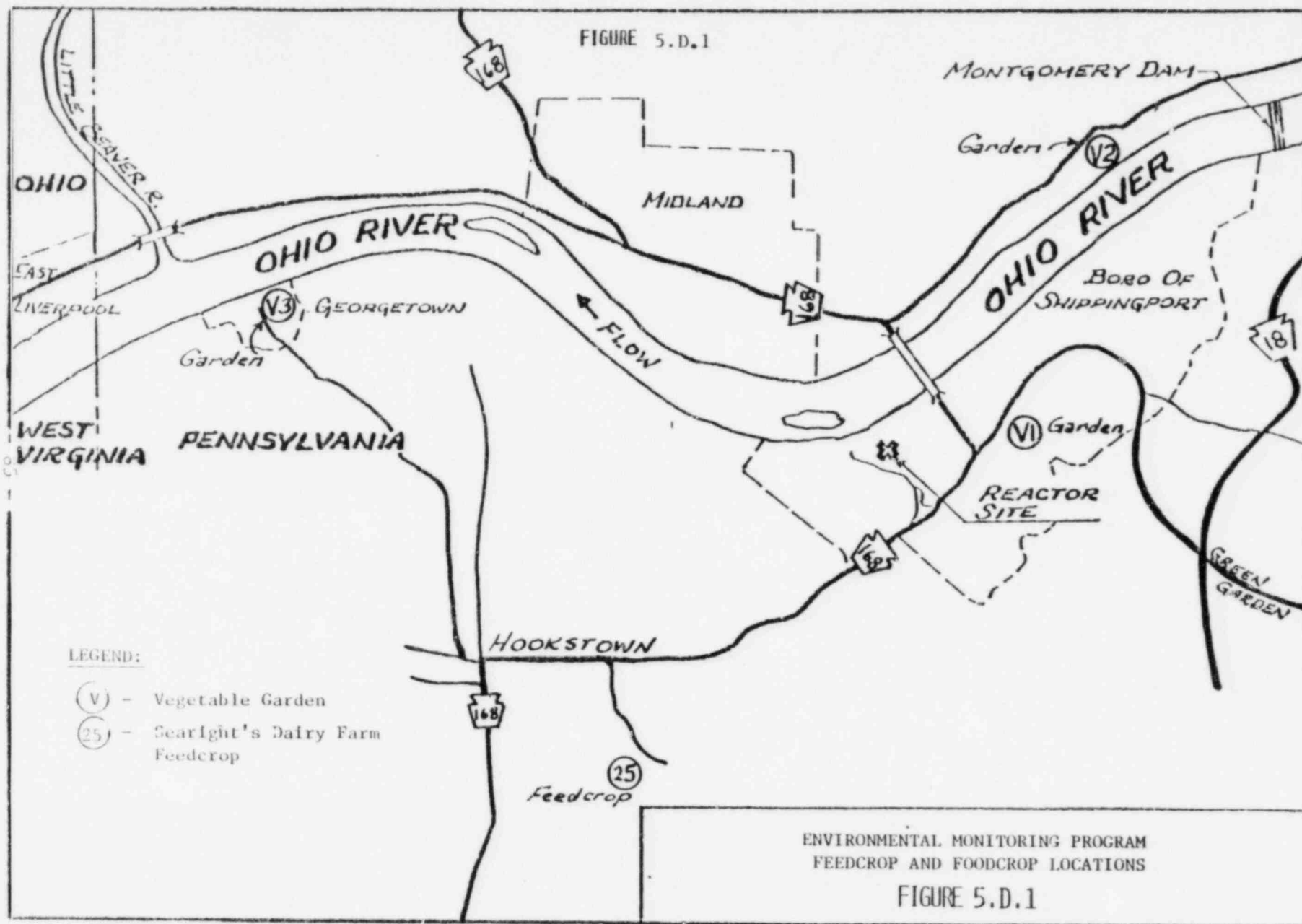
Representative samples of cattle feed are collected monthly from the nearest dairy (Searight). See Figure 5.D.1. Each sample is analyzed by gamma spectrometry. The monthly samples are composited into a quarterly sample which is analyzed for Sr-90.

Foodcrops (vegetables) were collected at garden locations during the summer of 1983. Leafy vegetables, i.e., cabbage, spinach, and lettuce were obtained from Shippingport, Georgetown, Industry, PA, and from Weirton, WV. All samples were analyzed for gamma emitters (including I-131 by gamma spectrometry).

b. Procedures

Gamma emitters, including I-131, are determined by scanning a dried, homogenized sample with the gamma spectrometry system. A Ge(Li) detector is utilized with this system.

Strontium 90 analysis for feedstuff is performed by a procedure similar to that described in V.C.2.



V. ENVIRONMENTAL MONITORING3. Results and Conclusions

A summary of results is provided in Table V.A.2. The predominant isotope detected was naturally occurring K-40 in both food and feed. Other activity is attributable to residuals from previous nuclear weapons tests or naturally occurring radionuclides. All results were consistent with (or lower than) those obtained in the pre-operational program. These data confirm that Shippingport Atomic Power Station and Beaver Valley Power Station did not contribute to radioactivity in foods and feeds in the vicinity of the site.

V. ENVIRONMENTAL MONITORINGE. Monitoring of Local Cow's Milk1. Description - Milch Animal Locations

During the seasons that animals producing milk (milch animals) for human consumption are on pasture, samples of fresh milk are obtained from these animals at locations and frequencies noted in Table V.A.1. This milk is analyzed for its radioiodine content calculated as Iodine-131. The analyses are performed within eight (8) days of sampling.

Detailed field surveys are performed during the grazing season to locate and enumerate milch animals within a five (5) mile radius of the site. Goat herd locations out to fifteen (15) miles are identified. Survey data for the most recent survey conducted in July, 1982 is shown in Figure 5.E.1.

2. Sampling Program and Analytical Techniques

a. Program

Milk was collected from three (3) reference dairy farms within a 10-mile radius of the site and from one (1) control location outside of the 10-mile radius. Additional dairies, which represent the highest potential milk pathway for radioiodine based on milch animal surveys and meteorological data were selected and sampled. These dairies are subject to change when more recent data (including census) indicate other locations are more appropriate. The location of each is shown in Figure 5.E.2 and described below.

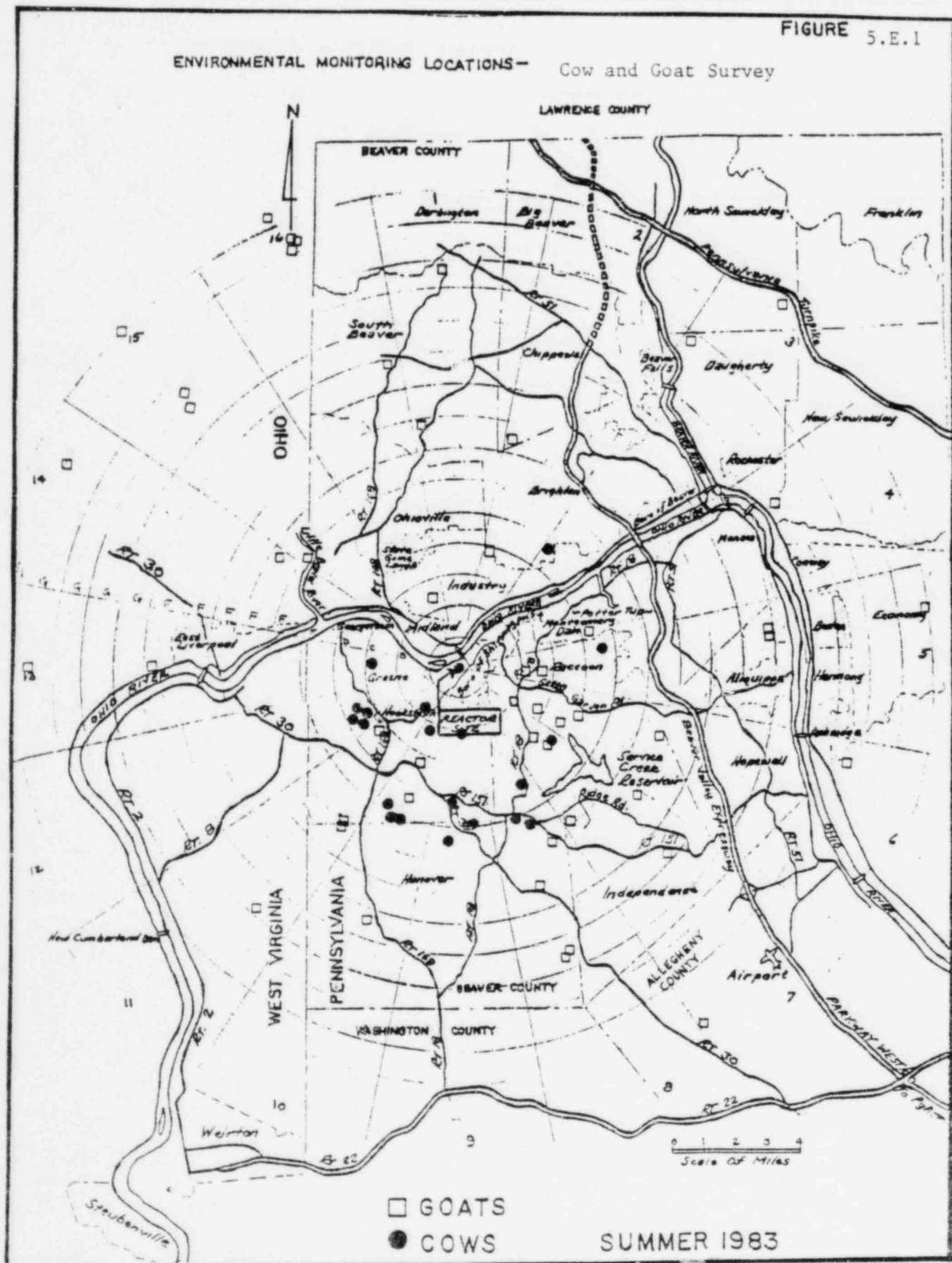
<u>Site</u>	<u>Dairy</u>	<u>Number of Milch Animals</u>	<u>Distance and Direction From Site</u>	<u>Collection Period</u>
25	Searight	41 Cows	2.1 miles-south/sw.	Jan. - Dec.
27	Brunton	105 Cows	7.3 miles-southeast	Jan. - Dec.
29A	Nichol	55 Cows	8.0 miles-northeast	Jan. - Dec.
96	Windsheimer	50 Cows	10.3 miles-south/sw.	Jan. - Dec.
61	Allison	33 Cows	3.2 miles-west/sw.	Jan.-June,Dec.
69	Collins	9 Goats, 1 Cow*	3.6 miles-southeast	Feb. - Dec.

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<u>Site</u>	<u>Dairy</u>	<u>Number of Milch Animals</u>	<u>Distance and Direction From Site</u>	<u>Collection Period</u>
98	Foxall (Hammond)	2 Goats*	2.9 miles-east	July - Nov.
100	Doughty	1 Cow*	2.8 miles-west/nw.	April - Dec.

*Milk Usage - Home Only.



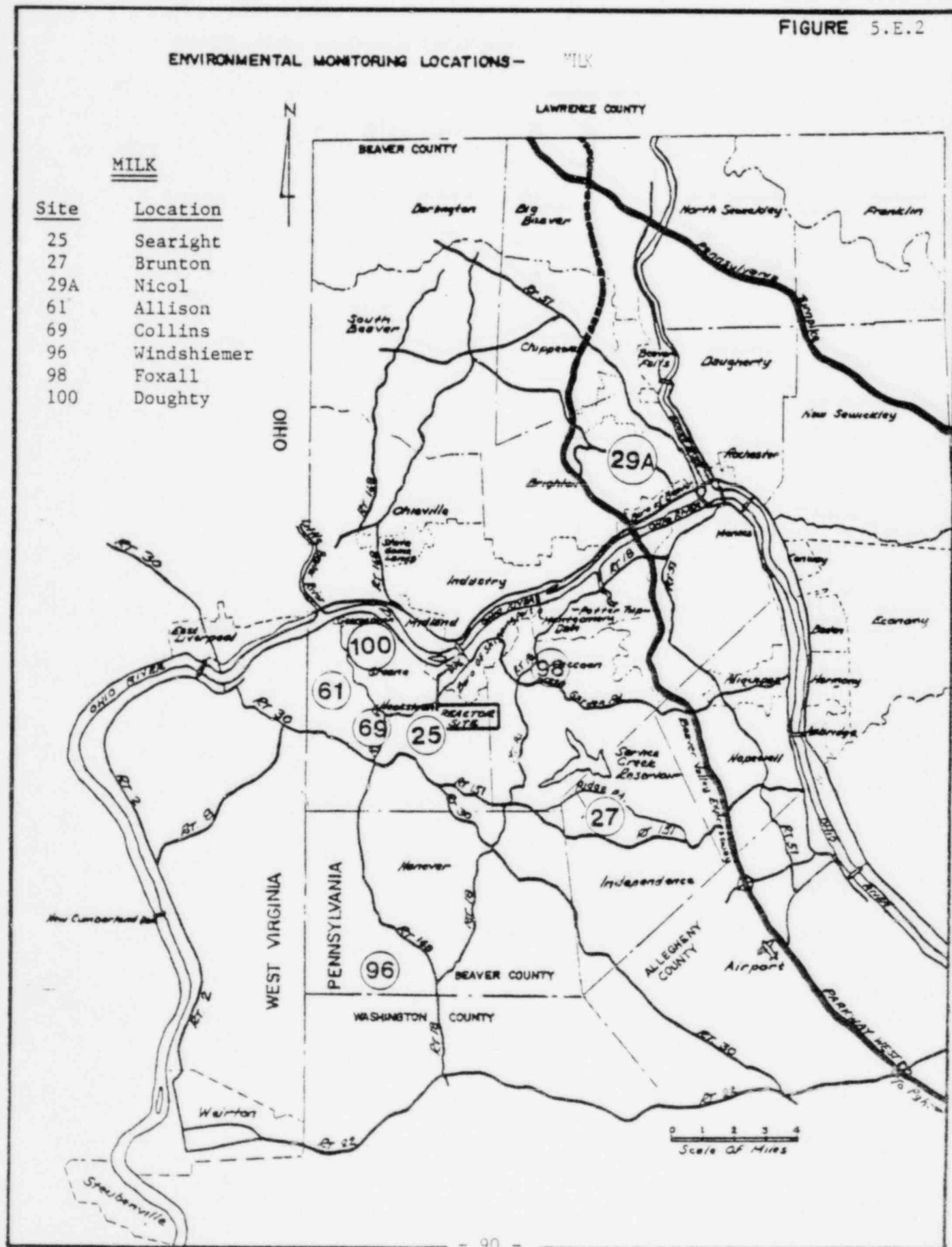
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FIGURE 5.E.2

ENVIRONMENTAL MONITORING LOCATIONS - MILK

MILK

<u>Site</u>	<u>Location</u>
25	Searight
27	Brunton
29A	Nicol
61	Allison
69	Collins
96	Windshiemer
98	Foxall
100	Doughty



V. ENVIRONMENTAL MONITORINGE. Monitoring of Local Cow's Milk (continued)

a. Program (continued)

The sample from Searight Dairy was collected and analyzed weekly for radioiodine using a procedure with a high sensitivity. Samples from each of the other selected dairies were collected monthly when cows are indoors, and bi-weekly when cows are grazing. This monthly or bi-weekly sample is analyzed for Sr-89, Sr-90, gamma emitters including Cs-137 (by Spectrometry) and I-131 (high sensitivity analysis).

b. Procedure

Radioiodine (I-131) analysis in milk was normally performed using chemically prepared samples and analyzed with a beta-gamma coincidence counting system.

Gamma emitters are determined by gamma spectrometry of a one liter Marinelli container of milk.

Strontium analysis of milk is similar to that of other foods (refer to V.C.2) except that milk samples are prepared by addition of Trichloroacetic Acid (TCA) to produce a curd which is removed by filtration and discarded. An oxalate precipitate is ashed for counting.

3. Results and Conclusions

A total of one hundred and fifty-two (152) samples were analyzed for I-131 during 1983. All I-131 activities in milk were below the minimum detectable level (0.3 pCi/l).

A total of one hundred and twenty-three (123) samples were analyzed by gamma spectrometry and for strontium. Both the Cs-137 and Sr-90 levels were within the normally expected range. The higher levels from China's nuclear tests which were experienced in other years were absent this year. Such tests were not performed during 1983. Based on analytical results, the operation of Beaver Valley Power Station and Shippingport Atomic Power Station did not contribute to any increase in airborne radioiodine in CY 1983.

V. ENVIRONMENTAL MONITORINGF. Environmental Radiation Monitoring1. Description of Regional Background Radiation Levels and Sources

The terrain in the vicinity of the Shippingport and Beaver Valley Power Station generally consists of rough hills with altitude variations of 300 to 400 feet. Most of the land is wooded.

The principal geologic features of the region are nearly flat-lying sedimentary beds of the Pennsylvania Age. Beds of limestone alternate with sandstone and shale with abundant interbedded coal layers. Pleistocene glacial deposits partially cover the older sedimentary deposits in the northwest. Most of the region is underlain by shale, sandstone, and some coal beds of the Conemaugh Formation. Outcrops of sandstone, shale, and limestone of the Allegheny Formation exist within the Ohio River Valley and along major tributary streams.

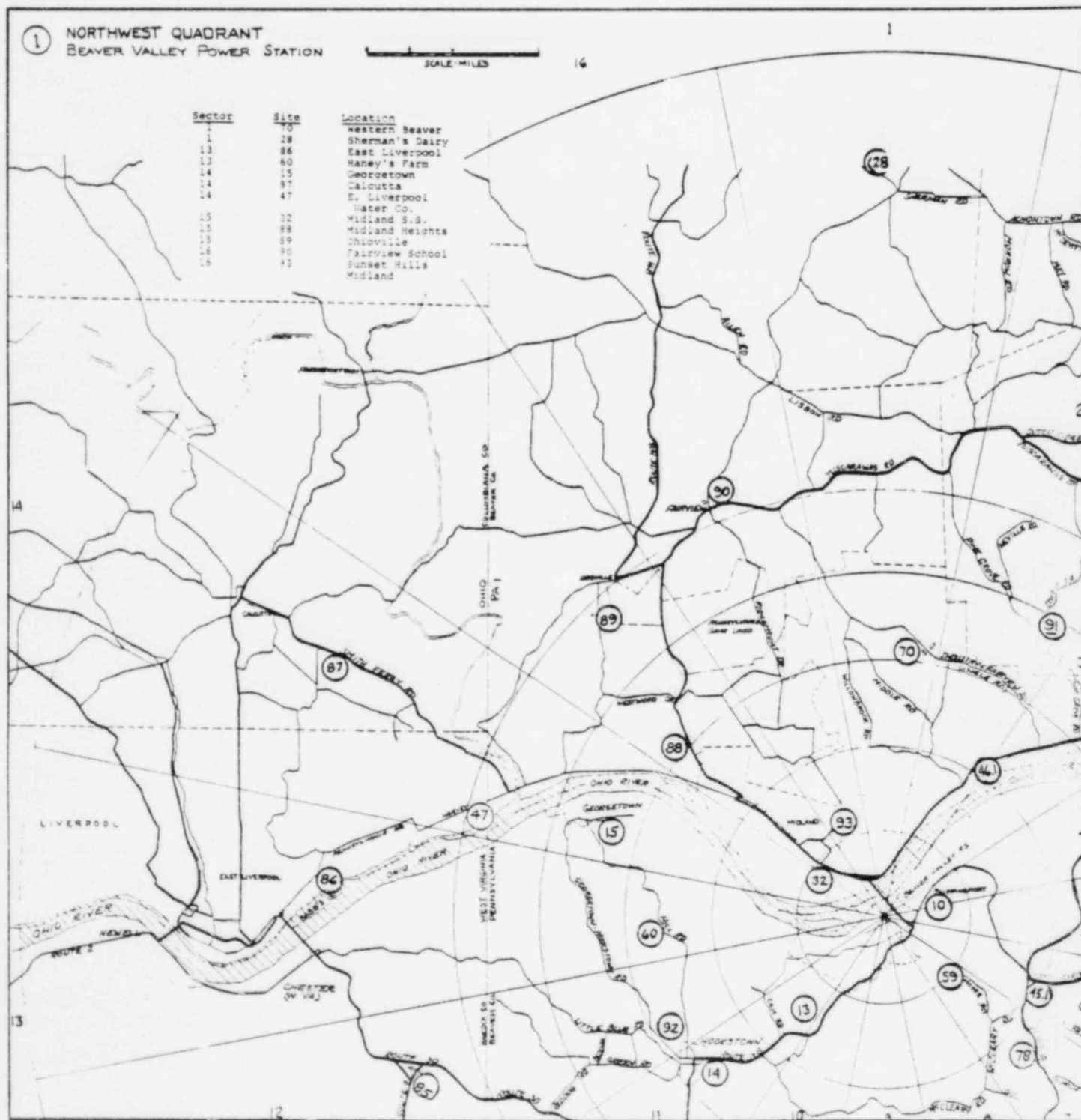
Based on surveys reported in previous annual reports, exposure rates ranged from 6 to 12 $\mu\text{R/hr}$. Results for 1983 indicated that background radiation continued in this range.

2. Locations & Analytical Procedures

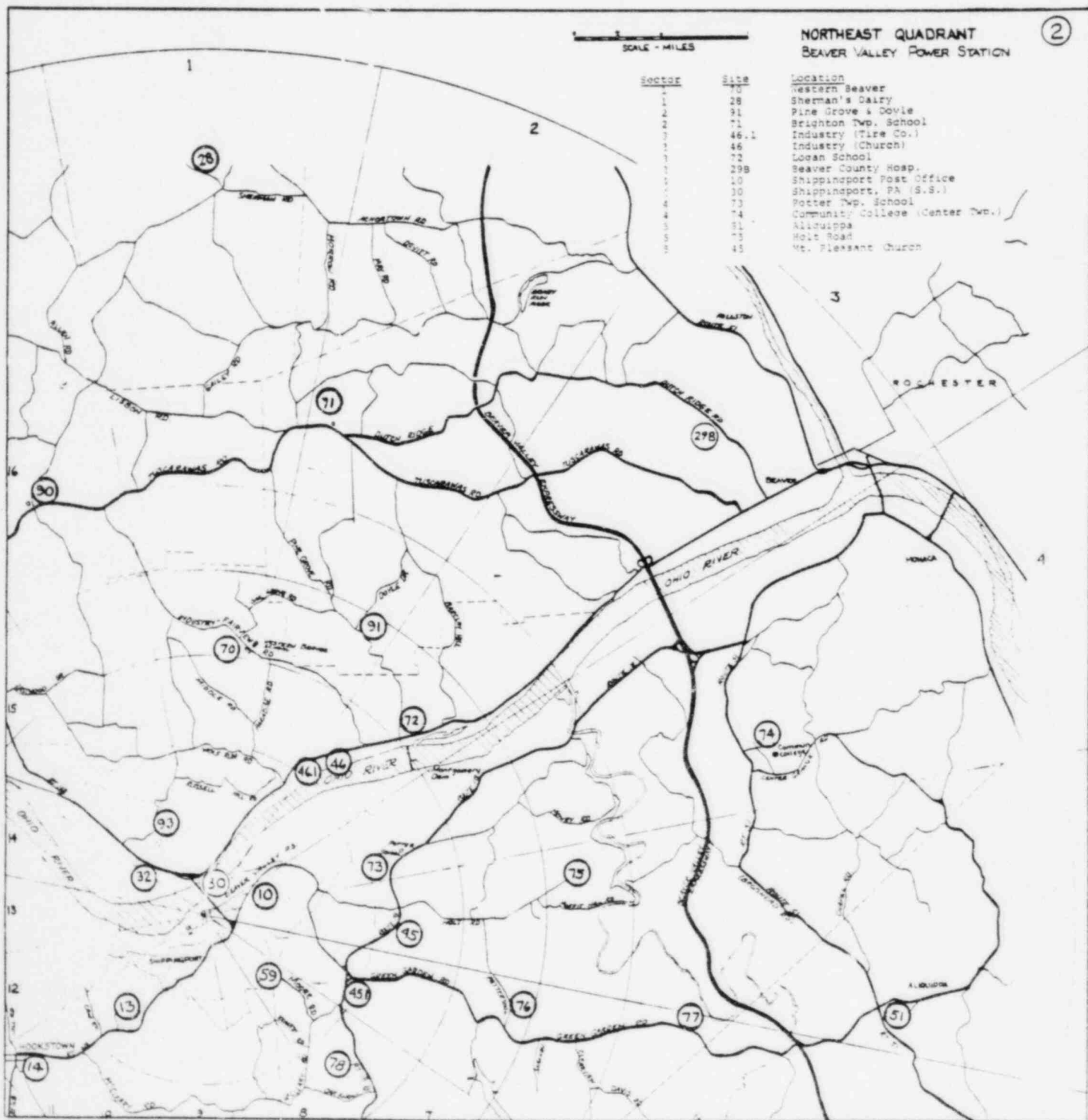
Ambient external radiation levels at the site were measured using thermoluminescent dosimeters (TLDs). There were three (3) types used in the Duquesne Light Company Radiological Environmental Monitoring Program. They are calcium sulphate dysprosium, CaSO_4 (Dy) in teflon matrix, lithium fluoride (LiF), and thulium activated calcium sulfate ($\text{CaSO}_4\text{:Tm}$).

The lithium fluoride TLDs were posted and analyzed by a laboratory of the Department of Energy (DOE) as an independent check of environmental radioactivity levels. The $\text{CaSO}_4\text{:Tm}$ TLDs were used as a back-up and as a QC program. In 1983 there were a total of forty-four (44) off-site environmental TLD locations. The locations of the TLDs are shown in Figures 5.F.1 thru 4. Comparisons of TLD results are presented in Table III.1.

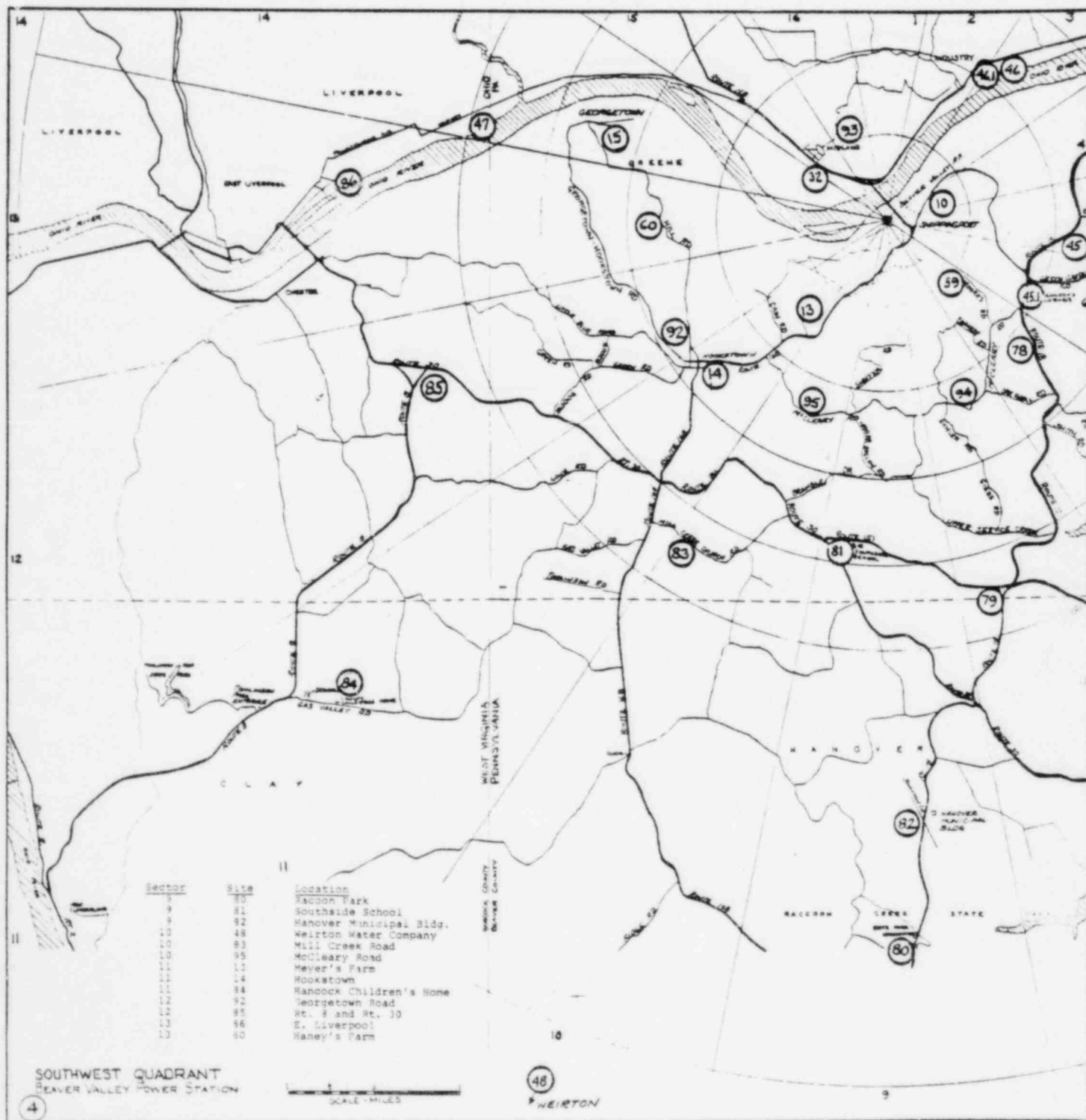
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V. ENVIRONMENTAL MONITORING2. Locations & Analytical Procedures (continued)

The lithium fluoride (LiF) TLDs used for environmental purposes are pre-selected and annealed at least 5 working days prior to use. The radiation dose accumulated from the anneal date to the date of posting is accounted for by utilizing background readings from five (5) TLD chips in conjunction with an average background correction factor. The calibration of the TLD reader is performed within 24 hours of processing the posted environmental TLDs. The environmental TLDs are processed after retrieval and a background correction is made to account for the background radiation accumulated from the date of retrieval to the date of processing.

The calcium sulfate (CaSO_4 :Dy) TLDs were annealed shortly before placing the TLDs in their field locations. The radiation dose accumulated in-transit between the field location and the laboratory was corrected by annealing control dosimeters shortly before the field dosimeters were removed from the field location, then shipping the freshly annealed control dosimeters with the exposed field dosimeters to the laboratory for readout at the same time. All dosimeters were exposed in the field in a special environmental holder. The dosimetry system was calibrated by reading calcium sulfate dosimeters which have been exposed in an accurately known gamma radiation field.

3. Results and Conclusions

Data obtained with the contractor TLD (CaSO_4 :Dy in teflon) during 1983 are summarized in Table V.A.2, and the quality control TLD results are listed in Table III.1.

The annual exposure rate of all off-site TLD's averaged 0.17 mR/day in 1983. As in previous years, there was some variation among locations and seasons as would be expected.

3. Results and Conclusions (continued)

In 1983, ionizing radiation dose determinations averaged approximately 62 mR for the year. This is comparable to previous years. There was no evidence of anomalies that could be attributed to the operation of either Beaver Valley Power Station or Shippingport Atomic Power Station. Three sets of TLDs of different types, each provided and analyzed by a separate laboratory, demonstrate good agreement and confirm that changes from natural radiation levels, if any, are negligible.

Lessons learned from the Three Mile Island incident indicated the need for more radiation monitors in all sectors surrounding the plant. Sixteen (16) Reuter-Stokes Pressurized Ion Chamber environmental radiation monitors were installed in 1982, one in each of the 16 sectors. Installation of the central processing controller into the Emergency Response Facility was expected to be completed in 1983 but is now expected to be completed in 1984 which will permit the monitors to be read out at a central location. Engineering is also in progress to upgrade and modify the BVPS meteorological system to meet requirements in U.S. NRC Regulatory Guide 1.23, Rev. 1, and U.S. NUREG-0654, Appendix 2. The Reuter Stokes radiation monitors and the upgraded meteorological system will be tied into a new computer network to help meet some of the requirements set forth in U.S. NRC NUREG-0654 (Criteria for Preparational Evaluation of Radiological Emergency Response Plans and Preparedness in Support of Nuclear Power Plants) and U.S. NRC NUREG-0737 (TMI Action Plan Requirements). The complete meteorological system modifications and new computer network which were scheduled to be operational by December 1983 are now expected to be operational by the end of the fourth BVPS #1 refueling outage.

V. ENVIRONMENTAL MONITORINGG. Monitoring of Fish1. Description

Fish collected near the site are generally scrap fish. During 1983, fish collected for the radiological monitoring program included carp, catfish, crappie bass, gizzard shad and bass.

2. Sampling Program and Analytical Techniques

a. Program

Fish samples are collected semi-annually in the New Cumberland pool of the Ohio River at the Beaver Valley and Shippingport effluent discharge points and upstream of the Montgomery Dam. The edible portion of each different species caught is analyzed by gamma spectrometry. Fish sampling locations are shown in Figure 5.G.1.

b. Procedure

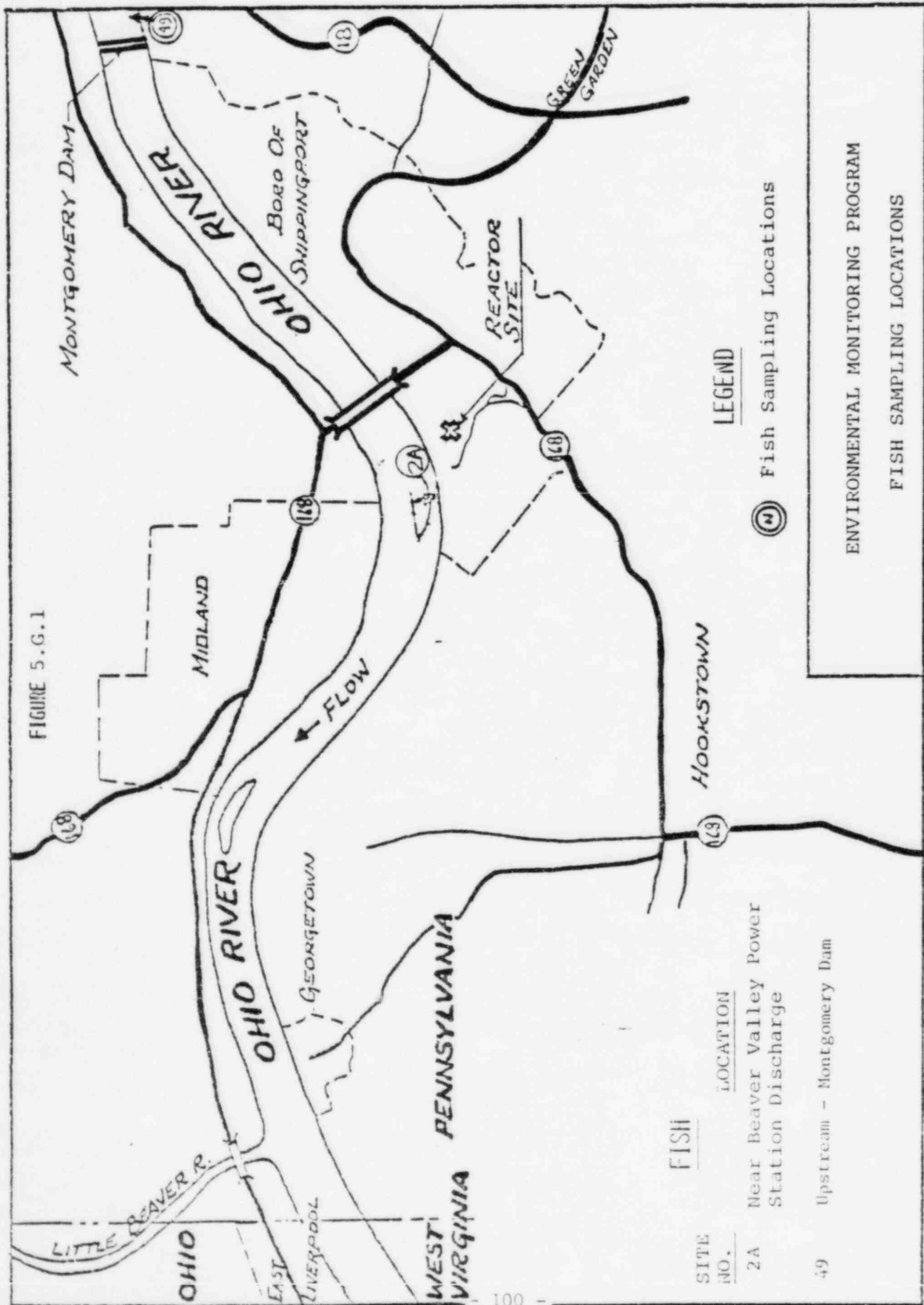
A sample is prepared in a standard tared 300 ml plastic bottle and scanned for gamma emitting nuclides with gamma spectrometry system which utilizes a Ge(Li) detector.

3. Results and Conclusions

A summary of the results of the fish monitoring data is provided in Table V.A.2. Five (5) fish were caught in April. Seven (7) more samples were caught in October. Except for naturally occurring K-40, the only gamma emitter which was detected in any samples was a trace of Cs-137 in one sample. Cesium-137 is a long lived fission product and some residual activity persists from previous weapons testing programs. This indicates that the operation of the Shippingport Atomic Power Station and the Beaver Valley Power Station has not resulted in radioactivity in fish in the Ohio River.

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FIGURE 5.G.1



V. ENVIRONMENTAL MONITORINGH. Monitoring of Surface, Drinking, and Well Waters1. Description of Water Sources

The Ohio River is the main body of water in the area. It is used by both the Beaver Valley and Shippingport plants for water make-up and receiving plant liquid effluents. In addition, river water is used for cooling purposes at the Shippingport Atomic Power Station and make-up for the cooling tower at the Beaver Valley Power Station.

Ohio River water is a source of water for some towns both upstream and downstream of the Beaver Valley and Shippingport plant sites. It is used by several municipalities and industries downstream of the site. The nearest user of the Ohio River as a potable water source is Midland Borough Municipal Water Authority. The intake of the treatment plant is approximately 1.5 miles downstream and on the opposite side of the river. The next downstream users are East Liverpool, Ohio, and Chester, West Virginia, which are approximately 6 and 7 miles downstream, respectively. The heavy industries in Midland, as well as others downstream use river water for cooling purposes. Some of these plants also have private treatment facilities for plant sanitary water.

Ground water occurs in large volumes in the gravel terraces which lie along the river, and diminishes considerably in the bedrock underlying the site. Normal well yields in the bedrock are less than 10 gallons per minute (gpm) with occasional wells yielding up to 60 gpm.

2. Sampling and Analytical Techniquesa. Surface (Raw River) Water

The sampling program of river water includes six (6) sampling points along the Ohio River. Raw water samples are normally collected at the East Liverpool (Ohio) Water Treatment Plant [River Mile 41.2] daily and composited into a monthly sample. Weekly grab samples are taken from the Ohio River at the following locations: Upstream of Montgomery Dam [River Mile 31.8]; at discharge from Shippingport Atomic Power Station [River Mile 34.8]; and near the discharge from the Beaver Valley Power Station [River Mile 35.0]. Two automatic river water samplers are at the following

locations: Upstream of Montgomery Dam [River Mile 29.6]; and at Crucible Steel's river water intake [River Mile 36.2]. The automatic sampler takes a 20 ml to 40 ml sample every 15 minutes and is collected on a weekly basis. The weekly grab samples and automatic water samples are composited into monthly samples from each location. In addition, a quarterly composite sample is prepared for each sample point.

The monthly composites are analyzed for gross alpha, gross beta, and gamma emitters. The quarterly composites are analyzed for tritium (H-3), strontium 89 (Sr-89), strontium 90 (Sr-90), and cobalt-60 (Co-60) (high sensitivity).

Locations of each sample point are shown in Figure 5.H.1.

b. Drinking Water (Public Supplies)

Drinking (treated) water is collected at both Midland (PA) and East Liverpool (OH) Water Treating Plants. An automatic sampler at each location collects 20-50 milliliters every 20 minutes. These intermittent samples are then composited into a weekly sample. The weekly sample from each location is analyzed by gamma spectrometry. The weekly samples are also analyzed for radioiodine (I-131).

Monthly composites of the weekly samples are analyzed for gross alpha, gross beta, and by gamma spectrometry. Quarterly composites are analyzed for H-3, Sr-89, Sr-90 and Co-60 (high sensitivity). Locations of each sample point are shown in Figure 5.H.1.

c. Ground Water

Grab samples were collected each quarter from each of four (4) well locations (see Figure 5.H.1) within four (4) miles of the site. These locations are:

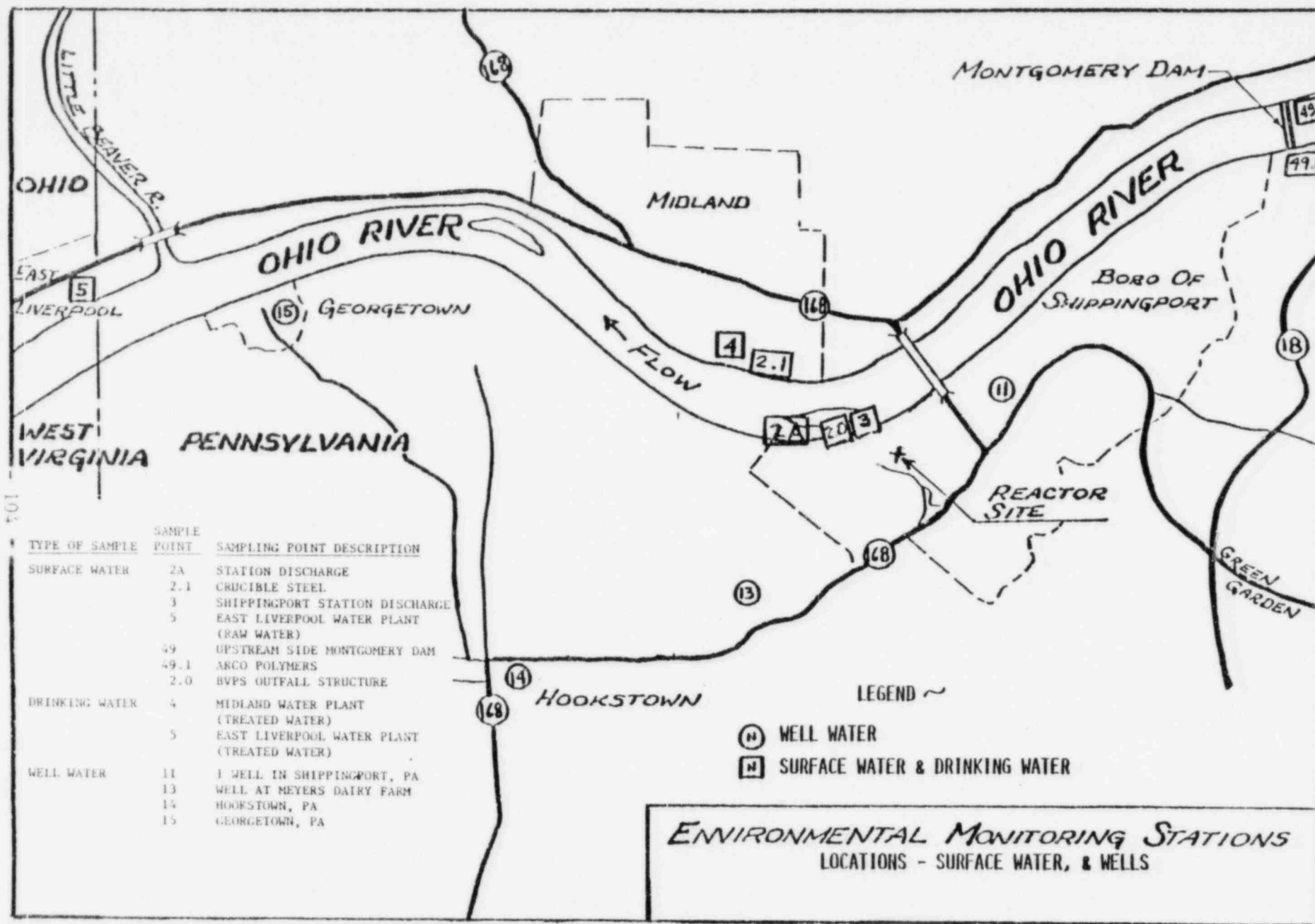
One (1) well at Shippingport, PA

One (1) well at Meyer's Farm (Hookstown, PA)

One (1) well in Hookstown, PA

One (1) well in Georgetown, PA

Each groundwater sample is analyzed for gross alpha, gross beta, tritium, and by gamma spectrometry.



V. ENVIRONMENTAL MONITORING2. Sampling and Analytical Techniques (continued)

d. Procedure

Gross alpha and gross beta activities are determined first by evaporating one liter of the sample on a hotplate. The residue is mounted and dried on a 2-inch stainless steel planchet. The sample is counted in a low background, gas flow proportional counter. Self-absorption corrections are made on the basis of sample weight.

Gamma analysis is performed on water sample by loading one liter of sample into a one liter marinelli container and counting on a Ge(Li) gamma spectrometry system.

Strontium-89 and 90 are determined on water samples by a procedure similar to that described in V.C.2 except that the leaching step is eliminated.

Cobalt-60 is determined with a sensitivity of 1 pCi/l by evaporating 2 liters of sample on a hotplate and transferring the residue to a 2-inch planchet. The planchet is counted on a Ge(Li) spectrometry system.

Tritium is determined in water samples by converting 2 ml of the sample to hydrogen and counting the activity in a 1 liter low level gas counter which is operated in the proportional range in anti-coincidence mode.

3. Results and Conclusions

A summary of results of all analyses of water samples (surface, drinking, and ground) are provided by sample type and analysis in Table V.A.2. These are discussed below.

a. Surface Water

A total of seventy-two (72) samples were each analyzed for gross alpha, gross beta, and gamma activity. Twenty-four (24) quarterly composited samples were analyzed for tritium (H-3) and radiostrontium (Sr-89 and Sr-90) as well as a high sensitivity analysis for Co-60.

V. ENVIRONMENTAL MONITORING3. Results and Conclusions (continued)

No Sr-90, Sr-89, Co-60, or gamma emitters were detected in surface water during CY 1983. All alpha and beta activities were within normal range.

The tritium levels in Beaver Valley Power Station and Shippingport Atomic Power Station outfall were elevated above preoperational levels in 1983, but none of these data suggests detectable increases over preoperational levels downstream of the station. The tritium activity is consistent with station data of authorized radioactive discharges and were well within limits permitted by NRC and NR license.

b. Drinking Water

A total of twenty-four (24) samples were analyzed for gross alpha and gross beta. All results were within a normal range.

A total of eight (8) samples were analyzed for tritium (H-3), radiostrontium (Sr-89 and Sr-90), and cobalt (Co-60). No Sr-89, Sr-90, or Co-60 were detected. The tritium data were within the preoperational range indicative of normal environmental levels.

A total of another one-hundred and three (103) samples were analyzed by gamma spectrometry. No gamma emitting radionuclides were detected by these analyses.

V. ENVIRONMENTAL MONITORING3. Results and Conclusions (continued)

A total of one hundred and four (104) samples were analyzed for radioiodine (I-131) using a highly sensitive technique. Trace levels of I-131 were measured in three (3) weekly samples collected at Midland. The results were slightly above the minimum detectable activity of 0.2 pCi/liter. The positive results could not be attributed to station discharges. The results may be attributed to expected variability in the analyses results of very low levels of activity.

c. Well Water

A total of sixteen (16) samples were each analyzed for gross alpha, gross beta, tritium and by gamma spectrometry. No alpha activity was detected in any of the samples. The gross beta and tritium data are within preoperational ranges.

d. Summary

The data from water analyses demonstrate that neither Beaver Valley Power Station nor Shippingport Atomic Power Station contributed a significant increase of radioactivity in local river, drinking or well waters. The few positive results which could be attributable to authorized releases from Beaver Valley Power Station and Shippingport Atomic Power Station are characteristic of the effluent. These results confirm that the station assessments, prior to authorizing radioactive discharges, are adequate and that the environmental monitoring program is sufficiently sensitive.

Further, the maximum detected activity, which was attributable to Beaver Valley Power Station, was only a small fraction (0.52%) of the concentration (averaged over a year) permitted by the Federal Regulations for water consumed by the public. The Ohio River further reduced this concentration prior to its potential use by members of the public.

V. ENVIRONMENTAL MONITORINGI. Estimates of Radiation Dose to Man1. Pathways to Man - Beaver Valley Power Stationa. Calculational Models - Beaver Valley Power Station

The radiation doses to man as a result of Beaver Valley operations were calculated for both gaseous and liquid effluent pathways using NRC computer codes XOQDOQ2, GASPAR, and LADTAP. Dose factors listed in Beaver Valley Power Station Environmental Technical Specifications were used to calculate doses to maximum individuals from radioactive noble gases in discharge plumes. Beaver Valley effluent data, based on sample analysis in accordance with the schedule set forth in Appendix B of the BVPS license, were used as the radionuclide activity input.

Each radionuclide contained in the semi-annual effluent report format of Regulatory Guide 1.21 was considered. Certain radionuclides which were not detected in the effluents were not included in dose calculations when the inventory of such nuclides available for discharge was judged to be negligible. As a result, only noble gases, radioiodines, strontium, and tritium were included as source terms based on the lower detectable limits of analysis (all sensitivities for analysis at Beaver Valley were equal to or better than required by the Beaver Valley license).

All gaseous effluent releases, including Auxiliary Building Ventilation, were included in dose assessments. The release activities are based on laboratory analysis. When the activity of noble gas was below detection sensitivity, either the inventory based on its MDL or an appropriate but conservative ratio to either measured activity of Kr-85 or Xe-133 was used. Meteorological data collected by the Beaver Valley Power Station Meteorology System was used as input to XOQDOQ2 which in turn provided input for GASPAR. Except when more recent or specific data was available, all inputs were the same as used in the Beaver Valley Power Station Environmental Statements or in Regulatory Guide 1.109. The airborne pathways evaluated were beta and gamma doses from noble gas plumes inhalation, the "cow-milk child", and other ingestion pathways.

V. ENVIRONMENTAL MONITORINGa. Calculational Models - Beaver Valley Power Station
(continued)

All potentially radioactive liquid effluents, including steam generator blowdown, are released by batch mode after analysis by gamma spectrometry using a GeLi detector. Each batch is diluted by cooling tower blowdown water prior to discharge into the Ohio River at the Beaver Valley Power Station outfall (River Mile 35.0). The actual data from these analyses are tabulated and used as the radionuclide activity input term in LADTAP. The maximum individual for liquid pathways is located at Midland. Except when more recent or specific data for the period is available, all other input to LADTAP are obtained from the Beaver Valley Power Station Environmental Statement or Regulatory Guide 1.109. Pathways, which were evaluated, are drinking water, fish consumption, shoreline recreation, swimming, and boating.

2. Results of Calculated Radiation Dose to Man - Beaver
Valley Power Station Liquid Releasesa. Liquid Pathway - Maximum Individual

The doses which are calculated, based on the model presented above in V.I.1, are summarized and compared to Beaver Valley Power Station license limits below. An additional breakdown of these doses by pathway and organ is provided in Table V.I.1. For these calculations, a hypothetical maximum individual(s) was located at Midland since this is the nearest location which significant exposure of a member of the public could potentially occur.

TABLE V.1.1
Radiation Dose to Maximum Individual^a, mrem/yr.
Beaver Valley Power Station - Liquid Releases

<u>PATHWAY</u>	<u>CRITICAL GROUP</u>	<u>USAGE FACTOR</u>	<u>SKIN</u>	<u>ORGAN</u>	<u>THYROID</u>	<u>BONE</u>	<u>WHOLE BODY</u>
Fish Consumption ^b	Adult	21.0 kg	N/A	0.0604 (Liver)	0.00227	0.0197	0.0276
Drinking Water ^c	Infant	510 l	N/A	0.0558 (Liver)	0.0696	0.00197	0.0535
Shoreline Activities	Teen	67 hr	0.00062	--	--	--	0.00057
TOTAL	MREM CRITICAL INDIVIDUAL		0.00062 (Teen)	0.0702 (Adult) (Liver)	0.0696 (Infant)	0.0254 (Child)	0.0613 (Adult)

DOSE TO INDIVIDUALS DURING 1983 FROM NATURAL RADIATION EXPOSURE

Ambient Gamma Radiation:	69 ^d
Radionuclides in Body :	18 ^e
Global Fallout :	4 ^e
TOTAL mrem :	91

^a Located at Midland Drinking Water Intake

^b Child - Usage Factor 6.9 kg/yr.

^c Adult - Usage Factor 730 l/yr.

^d Pre-operational average ambient gamma radiation

^e National Academy of Sciences, "The Effects on Populations of Exposure to Low Levels of Ionizing Radiation", BEIR Report, 1972.

SECTION V - I

DUQUESNE LIGHT COMPANY
1983 Annual Radiological Environmental ReportV. ENVIRONMENTAL MONITORING2. Results of Calculated Radiation Dose to Man - Beaver Valley Power Station Liquid Releases (continued)

Actual Doses (mrem/yr.) - Calculated Using Site Effluents Appendix I * Analysis Dose - Calculated Using NRC Model Effluents Regulatory Limit Doses - NRC Staff Guidelines RM50-2

	Calculated (1.21 Re- ported Re- leases)	Appendix I Report (Con- servative Non-accident Doses)	RM50-2 (Re. Limit w/o Cost/Benefit Analysis)	Ratio of Calculated Dose vs. Reg. Limit
<u>TOTAL BODY</u>				
Adult	0.0613	2.78	5.0	0.01226
Teen	0.0341	0.712	5.0	0.00682
Child	0.0416	Not Reported	5.0	0.00832
Infant	0.0535	Not Reported	5.0	0.01070
<u>ANY ORGAN</u>				
Adult	0.0702 (Liver)	Not Reported	5.0	0.01404
Teen	0.0551 (Liver)	Not Reported	5.0	0.01102
Child	0.0676 (Liver)	Not Reported	5.0	0.01352
Infant	0.0696 (Thyroid)	Not Reported	5.0	0.01392

Maximum Total Body Dose - Capsule Summary

	mrem
1983 Calculated	0.0613
Appendix I Estimated	2.78
Final Environmental Statement	0.112

Thyroid Dose - (Largest Expected Organ Dose)

1983 Calculated	0.0702
Final Environmental Statement	0.96

* 10 CFR 50 Appendix I

V. ENVIRONMENTAL MONITORING2. Results of Calculated Radiation Dose to Man - Beaver Valley Power Station Liquid Releases (continued)b. Population Doses

The 1983 calculated dose to the entire population of almost 4 million people within 50 miles of the plant was:

	<u>Man-Millirem</u>	<u>Largest Isotope Contributors</u>		
TOTAL BODY	398.0	H ₃	392	mrem
		Cs134	3.03	mrem
		Cs137	1.81	mrem
THYROID	437.0	H ₃	392	mrem
		I131	42.9	mrem

3. Airborne Pathway - (Beaver Valley Power Station)

The doses to the public for Beaver Valley Power Station airborne radioactive effluents during 1983 are provided in Table V.I.2. They include the contribution of all pathways. Tritium is the primary radionuclide contribution to these doses. The data demonstrate compliance with 10CFR50, Appendix I design objective limits.

4. Conclusions - (Beaver Valley Power Station)

Based upon the estimated dose to individuals from the natural background radiation exposure in Table V.I.1., the incremental increase in total body dose to the 50-mile population (4 million people), from the operation of Beaver Valley Power Station - Unit No. 1, is less than 0.0002% of the annual background.

The calculated doses to the public from the operation of Beaver Valley Power Station - Unit No. 1 are below 10CFR50, Appendix I design objectives, and resulted in only a small incremental dose to that which area residents already received as a result of natural background. The doses constituted no meaningful risk to the public.

TABLE V.I.2
Results of Calculated Radiation Dose to Man (1983)
Beaver Valley Power Station - Airborne Radioactivity

ORGAN	MAXIMUM EXPOSURE INDIVIDUAL, mrem	APPENDIX I* DESIGN OBJECTIVE mrem	PERCENT OF APPENDIX I	50-MILE POPULATION DOSE man rem
TOTAL BODY	0.0428	5	0.86%	0.158
SKIN	0.108	15	0.72%	0.492
LUNG	0.0416	--	--	0.171
THYROID	2.88	--	--	4.24

* 10CFR50, Appendix I

V. ENVIRONMENTAL MONITORING5. Dose Pathways to Man - Shippingport Atomic Power Station

The radiation doses to man as a result of operations at the Shippingport Atomic Power Station during 1983 were calculated for gaseous effluent pathways and liquid effluent pathways.

Effluent monitoring at the Shippingport Station during 1983 has shown that the radioactivity releases were substantially below the Federal radioactivity concentration guides. The environmental monitoring program has demonstrated that the radiation exposure to the general public from the Shippingport Station operations was too low to measure and could only be estimated with the calculational model described below using measured or estimated effluent radioactivity data.

a. Calculational Models - Shippingport Atomic Power Station

The radiation doses to man from Shippingport Atomic Power Station operations were estimated using calculational models recommended by the International Commission on Radiological Protection (ICRP Publ. 2, 1959) and employ the general guidelines of the Nuclear Regulatory Commission (Regulator Guide 1.109) established to maintain compliance with 10CFR50 Appendix I.

The air dose pathways considered were inhalation, immersion in gaseous and suspended particulate activity, and the ingestion of food and milk produced in the Shippingport vicinity. It was conservatively assumed that food products consumed by the public were produced in the Shippingport area throughout CY 1983. The maximum potentially exposed individual for the air pathways was located at the site boundary. It was conservatively assumed that the maximum individual resides continually at the site boundary.

The maximum potentially exposed individual for liquid pathways was located at Midland, PA. The liquid dose pathways considered were drinking water, fish consumption, shoreline recreation, swimming and boating.

5. Dose Pathways to Man - Shippingport Atomic Power Station
(continued)

Modeling parameters and usage factors used in the pathway calculations were consistent with values recommended by the Nuclear Regulatory Commission (NRC Regulatory Guide 1.109). The population distribution within 50 miles of the site was based on census data as provided in the LWBR Program Environmental Impact Statement (ERDA 1541). Furthermore, the air pathway calculation employed site-specific meteorological and wind direction data.

6. Results and Conclusions - Shippingport Atomic Power Station

Evaluation of the radiation dose-to-man calculations for the effluents show that the maximum annual radiation exposure potentially received by an individual residing at the site boundary is less than 0.1 mrem. The maximum dose to an individual is well below the 10CFR50 Appendix I dose limits. Furthermore, the radiation exposure to the entire population of 4 million persons with 50 miles of the Shippingport Station was less than 1 person-rem. This dose is negligible compared to the typical general use of approximately 360,000 person-rem received by all individuals from typical background radiation.

In conclusion, the radiation exposure received from the Shippingport Station during CY 1983 by any member of the general public is a very small fraction of the background radiation and has, therefore, no significant effect on the general public.

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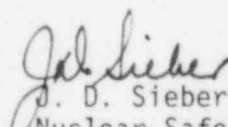
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Very truly yours,


J. D. Sieber, Manager
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