

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1): Oyster Creek, Unit 1										DOCKET NUMBER (2): 0 5 0 0 0 2 1 9					PAGE (3): 1 OF 4		
TITLE (4): Isolation Condenser Piping Leak Near Weld Joint																	
EVENT DATE (5):			LER NUMBER (6):				REPORT DATE (7):			OTHER FACILITIES INVOLVED (8):							
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME(S)				DOCKET NUMBER(S)				
0	3	2	8	4	8	4	0	0	5	0	0	0	5	0	0	0	
OPERATING MODE (9): N		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5. (Check one or more of the following) (11):															
POWER LEVEL (10): 01010		20.402(b)				20.406(c)				50.73(a)(2)(iv)				73.71(b)			
		20.406(a)(1)(i)				50.36(e)(1)				50.73(a)(2)(v)				73.71(c)			
		20.406(a)(1)(ii)				50.36(e)(2)				50.73(a)(2)(vi)				OTHER (Specify in Abstract below and in Text, NRC Form 305A)			
		20.406(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(vii)(A)							
		20.406(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(vii)(B)							
		20.406(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(ix)							
LICENSEE CONTACT FOR THIS LER (12):																	
NAME: John M. Charterina										TELEPHONE NUMBER: AREA CODE: 610 9 917 11-4161913							
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13):																	
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC							
B	I	- I P I S I F	X	91919	N												
SUPPLEMENTAL REPORT EXPECTED (14):										EXPECTED SUBMISSION DATE (15):		MONTH	DAY	YEAR			
X YES (If yes, complete EXPECTED SUBMISSION DATE)										NO		0	6	3	0	8	4
ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16):																	

See Page 2

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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)  Oyster Creek, Unit 1	DOCKET NUMBER (2)  0500021984-005-000204	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		

TEXT (If more space is required, use additional NRC Form 368A's) (17)

ABSTRACT

The shell side of both Isolation Condensers (NE01-A, NE01-B) was hydrolayzed and coated to protect against corrosion. As part of post-maintenance testing, the tube side of the condensers was hydrostatically tested to verify tube integrity. The test was performed by isolating the steam and condensate piping associated with each condenser, then pressurizing that portion of piping, and checking for leaks. Isolation Condenser NE01-B was tested first, with no apparent leakage. However, the hydro test on Isolation Condenser NE01-A was never completed.

On Thursday, March 22, 1984, during the hydro test being performed on Isolation Condenser NE01-A, a leak was noticed near an insulated elbow on the condensate piping, downstream of the condenser. The insulation was removed to reveal a crack near weld NE-2-12. The weld area was then examined ultrasonically, which pinpointed the existence of a through-wall crack approximately 5 inches long. The indications are that the failure may be Intergranular Stress Corrosion Cracking. Plans have been developed to inspect more of the welds in this system, including those associated with the second Isolation Condenser NE01-B.

DATE OF OCCURRENCE

The source of the leak was found on Thursday, March 22, 1984.

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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FACILITY NAME (1)  Oyster Creek, Unit 1	DOCKET NUMBER (2)  0 5 0 0 0 2 1 9	LER NUMBER (6)			PAGE (3)		
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TEXT (If more space is required, use additional NRC Form 366A's) (17)

IDENTIFICATION OF OCCURRENCE

Weld NE-2-12, associated with the piping from Isolation Condenser NE01-A, failed causing 5 inch and 1.650 inch through-wall cracks.

This event is considered to be a Reportable Occurrence as defined in 10CFR 50.73 (a)(2)(ii).

CONDITIONS PRIOR TO OCCURRENCE

The plant was shutdown with the mode switch in REFUEL and all fuel removed from the vessel.

DESCRIPTION OF OCCURRENCE

On Thursday, March 22, 1984, during a hydro test being performed on Isolation Condenser NE01-A, a leak was noticed near an insulated elbow on the condensate piping at a pressure lower than operating pressure. Removal of the insulation uncovered leakage weeping out near a weld. Further non-destructive examination using ultrasonics revealed the existence of 5 inch and 1.650 inch through-wall cracks.

APPARENT CAUSE OF OCCURRENCE

The indications are that the cause may be Intergranular Stress Corrosion Cracking. Plans are being made to retrieve a sample of the failed area and perform laboratory analysis to verify the failure mechanism.

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

ANALYSIS OF OCCURRENCE AND SAFETY ASSESSMENT

The safety significance of this event is considered minimal since the reactor was in a shutdown condition with no fuel in the core and the Isolation Condenser System was not required.

An investigation is being conducted to determine the safety significance of this event, had the plant been in operation. One of the topics being investigated is the possibility of a high energy break affecting the power source to its related loop isolation valves. The results of this investigation will be addressed in a supplemental report.

CORRECTIVE ACTION

Presently, steps are being taken to inspect the entire Isolation Condenser System including both loops on the steam side, as well as the condensate side, to determine the integrity of the remaining weld joints. Samples of the failed area are to be taken and analyzed to determine the cause of the failure. Only then can a program be developed to formulate the corrective action required. Further information will be submitted once the ongoing investigations are completed and analyzed.



**GPU Nuclear Corporation**

Post Office Box 388  
Route 9 South  
Forked River, New Jersey 08731-0388  
609 971-4000  
Writer's Direct Dial Number:

May 1, 1984

U.S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555

Dear Sir:

Subject: Oyster Creek Nuclear Generating Station  
Docket No. 50-219  
Licensee Event Report  
Reportable Occurrence No. 50-219/84-005

This letter forwards one (1) copy of a Licensee Event Report (LER) to report Reportable Occurrence No. 50-219/84-005 in compliance with 10CFR50.73.

Very truly yours,

Peter B. Fiedler  
Vice President and Director  
Oyster Creek

PBF:dam  
Enclosures

cc: Dr. Thomas E. Murley, Administrator  
Region I  
U.S. Nuclear Regulatory Commission  
631 Park Avenue  
King of Prussia, PA 19406

NRC Resident Inspector  
Oyster Creek Nuclear Generating Station  
Forked River, NJ 08731

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