

TENNESSEE VALLEY AUTHORITY
CHATTANOOGA, TENNESSEE
37401



August 19, 1974



Mr. John F. O'Leary, Director
Directorate of Licensing
Office of Regulation
U.S. Atomic Energy Commission
Washington, DC 20545

Dear Mr. O'Leary:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 2 -
DOCKET NO. 50-260 - FACILITY OPERATING LICENSE DPR-52 - ABNORMAL
OCCURRENCE REPORT BFAO-50-260/743W

The enclosed report is to provide details concerning malfunction of
RHR pump discharge pressure switch PS-74-19A and is submitted in
accordance with Appendix A to Regulatory Guide 1.16, Revision 1,
October 1973. This event occurred on Browns Ferry Nuclear Plant
unit 2 on August 9, 1974.

Very truly yours,

TENNESSEE VALLEY AUTHORITY

E. F. Thomas
Director of Power Production

Enclosure
CC (Enclosure):

Mr. Norman C. Moseley, Director
Region II Regulatory Operations Office, USAEC
230 Peachtree Street, NW., Suite 818
Atlanta, Georgia 30303

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inquiry*

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ABNORMAL OCCURRENCE REPORT

Report No.: BFAO-50-260/743W
Report Date: August 19, 1974
Occurrence Date: August 9, 1974
Facility: Browns Ferry Nuclear Plant unit 2

Identification of Occurrence

Malfunction of RHR pump discharge pressure switch PS-74-19A.

Conditions Prior to Occurrence

Reactor was operating at 800 psig and about 1-percent reactor power in the hot functional phase of the startup program.

Description of Occurrence

During routine surveillance testing, RHR pump discharge pressure switch PS-74-19A was found to be inoperative.

Designation of Apparent Cause of Occurrence

An inspection of the switch internals revealed that the actuator arm for the switch contacts had become disoriented and wedged between the bellows plunger and the adjustment nut. There is no known cause to explain the physical disorientation of the switch actuator arm.

Analysis of Occurrence

The redundant switch, PS-74-19B, was functionally tested and found to operate satisfactorily and would have performed its intended function. These switches are arranged in a 1 out of 2 logic and provide ADS permissive upon coincidence with low reactor water level, high drywell pressure, and a time delay of 120 seconds.

Corrective Action

A new switch was calibrated and placed in service. The switch was functionally tested and the associated logic performed as intended.

Failure Data

Static "O" Ring Pressure Switch
Catalog No. 5N-AA3-X7PP
Range 10 to 240 psig
S/N 70-10-1172

This mode of failure has not occurred previously for this equipment.