

LICENSEE EVENT REPORT (LER)

APPROVED OMS NO. 3180-0104
EXPIRES 8/31/85FACILITY NAME (1)
Peach Bottom Atomic Power Station - Unit 3DOCKET NUMBER (2)
0 5 0 0 0 2 7 8PAGE (3)
1 OF 03

TITLE (4)

Reactor Vessel Heatup Rate Exceeded 100 degrees Fahrenheit per hour.

EVENT DATE (5)
MONTH DAY YEAR
0 1 24 8 4 8 4

LER NUMBER (6)

SEQUENTIAL NUMBER

REVISION NUMBER

REPORT DATE (7)

MONTH DAY YEAR

0 1 0 5 0 3 8 4

OTHER FACILITIES INVOLVED (8)

FACILITY NAMES

DOCKET NUMBER(S)

0 5 0 0 0 0

0 5 0 0 0 0

OPERATING
MODE (9)

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5. (Check one or more of the following) (11)

POWER
LEVEL
(10)

20.402(b)

20.406(a)(1)(i)

20.406(a)(1)(ii)

20.406(a)(1)(iii)

20.406(a)(1)(iv)

20.406(a)(1)(v)

20.406(a)(1)(vi)

20.406(e)

60.38(a)(1)

60.38(a)(2)

60.73(a)(2)(i)

60.73(a)(2)(ii)

60.73(a)(2)(iii)

60.73(a)(2)(iv)

60.73(a)(2)(iv)

60.73(a)(2)(v)

60.73(a)(2)(vi)

60.73(a)(2)(vii)(A)

60.73(a)(2)(vii)(B)

60.73(a)(2)(viii)

60.73(a)(2)(ix)

73.71(b)

73.71(a)

73.71(a)

OTHER (Specify in Abstract

below and in Text, NRC Form

364A)

LICENSEE CONTACT FOR THIS LER (12)

NAME

B. L. Clark, Senior Engineer - Special Projects

TELEPHONE NUMBER

AREA CODE

2 1 5

8 4 1 1 - 5 0 1 1 7

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14)

EXPECTED
SUBMISSION
DATE (15)

MONTH DAY YEAR

YES (If yes, complete EXPECTED SUBMISSION DATE)

X

NO

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

Abstract 3-84-02

Technical Specification 3.6.A.1 states that average rate of reactor coolant temperature change during normal heatup or cooldown shall not exceed 100 degrees Fahrenheit increase (or decrease) in any one hour period.

During a unit startup, a calculational error resulted in a reactor heatup rate of 111 degrees Fahrenheit per hour. Upon discovery of the error, the heatup rate was reduced immediately by the reactor operator to bring the unit to less than its Technical Specification limit of 100 degrees Fahrenheit per hour.

General Electric has evaluated the excess vessel heatup rate and concluded that it had a minimal effect on the reactor pressure vessel.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1) Peach Bottom Atomic Power Station Unit 3	DOCKET NUMBER (2) 050002718	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		84	01 d2	011	02	OF	03

TEXT (If more space is required, use additional NRC Form 366A) (17)

Description of the Event:

During a startup on Unit 3, reactor operator trainees were recording and calculating the reactor coolant temperature changes per Surveillance Test 9.12 (Reactor Vessel Temperatures). The temperatures recorded were the reactor pressure vessel drain temperature, the Recirculation Loop A temperature, and the Recirculation Loop B temperature at 15 minute intervals during the heatup. The Reactor Operator's review of the Surveillance Test data identified the calculation errors. Due to the errors, the heatup rate exceeded its Technical Specification limit of 100 degrees Fahrenheit in a 1-hour period. Applicable Technical Specification reference 3.6.A.1, states that the average rate of reactor coolant temperature change during normal heatup or cooldown shall not exceed 100 degrees Fahrenheit increase (or decrease) in any 1-hour period.

Consequences of the Event:

Heatup rates far greater than 100 degrees Fahrenheit per hour were considered in the original reactor vessel design analysis. This transient has been evaluated by GE and determined to have imposed only a negligible increase in the usage factor for the most limiting reactor pressure vessel component.

Cause of the Event:

The cause of the event was personnel error.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Peach Bottom Atomic Power Station Unit 3	DOCKET NUMBER (2) 05000278	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		84	002	01	03	OF	03

TEXT (if more space is required, use additional NRC Form 368a) (17)

Corrective Actions:

When it was realized that the heatup rate exceeded its Technical Specification limit of 100 degrees Fahrenheit in a 1-hour period, the reactor operator acted immediately to reduce the heatup rate.

The Reactor Operator in charge of operating the reactor during this event has been advised of his responsibility in maintaining heat-up rates under Technical Specification limits. Additionally, a station superintendent has issued a memo to all licensed operators to ensure that the intent of the Technical Specification heatup rate requirement is understood and to emphasize the use of the diverse instrumentation available in the control room as a cross check on reactor heatup rate.

PHILADELPHIA ELECTRIC COMPANY

2301 MARKET STREET

P.O. BOX 8699

PHILADELPHIA, PA. 19101

(215) 841-4000

May 3, 1984

Docket No. 50-278

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

SUBJECT: Licensee Event Report

This LER concerns the Unit 3 reactor vessel heatup rate exceeding its Technical Specification limit of 100 degrees Fahrenheit increase in a 1-hour period, while in the startup mode.

Reference:	Docket No. 50-278
Report Number:	3-84-02
Revision Number:	01
Event Date:	January 24, 1984
Report Date:	May 3, 1984
Facility:	Peach Bottom Atomic Power Station RD #1, Box 208, Delta, PA 17314

This LER is submitted pursuant to the requirements of 10 CFR 50.73(a)(2)(i)(B). This revised portion of this LER is identified by a vertical bar in the margin.

Very truly yours,



W. T. Ullrich
Superintendent
Nuclear Generation Division

cc: Dr. Thomas E. Murley, Administrator
Region I, USNRC

Mr. A. R. Blough, Site Inspector

IE22
11