

Jersey Central Power & Light Company

MADISON AVENUE AT HUNCH BOWL ROAD • MORRISTOWN, N. J. 07960 • 539-6111

March 17, 1971

Dr. Peter A. Morris, Director
Division of Reactor Licensing
United States Atomic Energy Commission
Washington D. C. 20545



Dear Dr. Morris:

Subject: Oyster Creek Station
Docket No. 50-219
Main Steam Isolation Valve Testing

The purpose of this letter is to advise you that the Main Steam Isolation Valves at our Oyster Creek Station were again tested for leakage on February 16-17, 1971 and to report the results of these tests. An opportunity to test the valves occurred because of a scheduled outage to inspect the turbine control valves.

The final results of these tests indicated the following leak rates:

NS03A -- Less than 0.1 SCFH NS04A = 3.6 SCFH

NS03B -- Less than 0.1 SCFH NS04B = 4.3 SCFH

These results are within the allowable Technical Specification leak rate of 11.1 Standard Cubic Feet Per Hour (SCFH) at 20 psig. NS03A and NS03B are the north and south valves, respectively, that are located inside the drywell. NS04A and NS04B are the north and south valves, respectively, that are located outside the drywell.

On February 16, 1971, the reactor vessel was pressurized to 20 psig in order to test the inner valves. The results showed that air was leaking through NS03B and that NS03A had no detectable leakage. Both of the inside valves were then cycled and retested and again NS03A had no detectable leakage and the measured leak rate of NS03B was approximately 380 SCFH. The reactor vessel was then vented, the main steam lines were flooded with water, and a 20 psig pressure decay test was conducted between the north valve pair in order to determine the leakage through NS04A. As reported above, this leakage was observed to be 3.6 SCFH. During this test, no water leakage was detected past either of the inside valves; and since the reactor vessel water level

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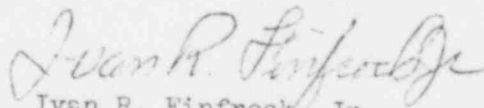
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was already raised for the valve testing, a 1000 psig hydrostatic test of the primary system was performed as the result of maintenance work done on the electromatic relief valves. Again, during this hydrostatic test, no water leakage was detected past the inside main steam isolation valves. Following a successful hydrostatic test, a pressure decay test was performed between the south pair of isolation valves and the leakage through NS04B was determined to be 4.3 SCFH. The steam lines above the inside isolation valves were then drained and NS03B was retested, and there was no detectable leakage through this valve with 20 psig air pressure in the reactor vessel.

The results of this testing indicate that the seating surfaces of NS03B, as well as the other valves, have remained in good condition, particularly the inside valves, since no water leakage was detected at 1000 psig water pressure and no air leakage was detected under standard test conditions. It is evident that NS03B requires a larger seating force than the other valves and close attention will be paid to the performance of this valve when it is tested again. In the meantime, we are confident this valve will perform its safety function since 850 to 1000 psig steam pressure is available to assist in seating the valve upon an isolation signal.

Very truly yours,



Ivan R. Finfrock, Jr.
Manager, Nuclear Generating Stations

IRF/pk

cc: Mr. R. W. Kirkman, Regional Director
Division of Compliance