

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

CON'T

60 61 DOCKET NUMBER 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | During a review of plant documentation, it was revealed that the data sheets concerning

0 3 | LLRT testing of the primary containment airlock, drywell equipment hatch, and drywell

0 4 | head access, PT-20.3, could not be located. Following this discovery, the airlock

0 5 | portion of this PT was satisfactorily completed on June 3, 1982. This event did not

0 6 | affect the health and safety of the public.

0 7 \_\_\_\_\_  
 0 8 \_\_\_\_\_ Technical Specifications 3.6.1.2, 3.6.1.3, and 6.9.1.9b  
 8

SYSTEM CODE S A 11		CAUSE CODE D 12		CAUSE SUBCODE Z 13		COMPONENT CODE Z Z Z Z Z Z 14		COMP. SUBCODE Z 15		VALVE SUBCODE Z 16	
LER/RO REPORT NUMBER 8 2 17		EVENT YEAR 8 2 21 22		SEQUENTIAL REPORT NO. 0 6 2 24 25 26		OCCURRENCE CODE 0 3 28 29		REPORT TYPE L 30		REVISION NO. 1 32	
ACTION TAKEN G 18		FUTURE ACTION Z 19		EFFECT ON PLANT Z 20		SHUTDOWN METHOD Z 21		HOURS 0 0 0 0 22		ATTACHMENT SUBMITTED Y 23	
NPRD-4 FORM SUB. Y 24		PRIME COMP. SUPPLIER Z 25		COMPONENT MANUFACTURER Z 9 9 9 26							

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 After further investigation, the airlock test report was located and determined to have  
1 1 been in the review process during the time of this event. The drywell equipment hatch  
1 2 and drywell head access data sheets could not be located; however, all tests have been  
1 3 completed satisfactorily. PT-20.3 was revised to include a better tracking system, the  
1 4 test specialist now keeps copies of all data sheets, and appropriate individuals have  
7 8 9 been counseled. No further action is planned for this event. 80

1 5 2 8 3 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

FACILITY STATUS (1) 5 (29) G (30) NA (31) B (32) Routine Review

% POWER (29) 0 0 0 10 12 13

OTHER STATUS (30) NA

DISCOVERY DESCRIPTION (32) Routine Review

ACTIVITY CONTENT  
RELEASED OF RELEASE

1 6 7 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

AMOUNT OF ACTIVITY (35) NA

LOCATION OF RELEASE (36) NA

PERSONNEL EXPOSURES									
NUMBER		TYPE		DESCRIPTION (39)					
1	7	0	0	0	(37)	Z	(38)	NA	

PERSONNEL INJURIES		DESCRIPTION (41)	
NUMBER			
1	0	0	0 (40)

NA

7 8 9 11 12  
LOSS OF OR DAMAGE TO FACILITY (43)  
TYPE DESCRIPTION  
1-1-1  
8405070207 840413  
PDR ADOCK 05000325  
S PDR

7 8 9 10

PUBLICITY

ISSUED DESCRIPTION (45)

NRC USE ONLY

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2 0 N 44 NA 68 69 80

NAME OF PREPARER Mary T. Allen

PHONE: 919-457-9521

LER ATTACHMENT - RO SUPPLEMENT TO 1-82-062

Facility: BSEP Unit 1

Event Date: June 3, 1982

During review of the PT status computer listing for PT-20.3, Local Leak Rate Testing of Containment Isolation, on June 3, 1982, it was noted that the test on the primary containment airlock was overdue. The computer listing indicated that the test should have been performed on April 13, 1982.

Although involved personnel remembered performing the test, supporting documentation could not be located. The test was immediately conducted and completed satisfactorily on June 3, 1982.

During a more extensive search, it was discovered that data sheets for tests H-1 (Drywell Equipment Hatch) and H-3 (Drywell Head Blank) were also missing. Since these hatches are inaccessible during operation, they were performed during the first outage of sufficient length and completed satisfactorily; (H-1 on July 19, 1982 and H-3 on July 18, 1982).

Through continuing investigation, the airlock test report was located, showing completion of the test on February 18, 1982. Since this document was reviewed by QA on June 28, 1982, it is concluded that, during the initial search, the document was in the review cycle. Since the H-1 and H-3 tests could not be located, it is determined that they were lost in the review chain.

To prevent recurrence, the test specialist now keeps a backup copy of all submitted data sheets. In addition, PT-20.3 was revised to include a better means of tracking test dates. Appropriate personnel have been counseled regarding review of test documentation for completeness and its prompt submittal to the vault.

PT-20.3 was in progress on Unit 2 at the time of the event and has since been completed. All tests were verified to have been performed.



Carolina Power & Light Company

APR 17 12:46

Brunswick Steam Electric Plant  
P. O. Box 10429  
Southport, NC 28461-0429  
April 13, 1984

FILE: B09-13510C  
SERIAL: BSEP/84-1017

Mr. James P. O'Reilly, Administrator  
U. S. Nuclear Regulatory Commission  
Region II, Suite 3100  
101 Marietta Street N.W.  
Atlanta, GA 30303

BRUNSWICK STEAM ELECTRIC PLANT, UNIT 1  
DOCKET NO. 50-325  
LICENSE NO. DPR-71  
SUPPLEMENT TO LICENSEE EVENT REPORT 1-82-62

Dear Mr. O'Reilly:

In accordance with Section 6.9.1.9b of the Technical Specifications for Brunswick Steam Electric Plant, Unit 1, the enclosed supplemental Licensee Event Report is submitted. The original report fulfilled the requirement for a written report within 30 days of a reportable occurrence and both are in accordance with the format set forth in NUREG-0161, July 1977.

Very truly yours,

C. R. Dietz, General Manager  
Brunswick Steam Electric Plant

RMP/sdl/LETKB1

Enclosure

cc: Mr. R. C. DeYoung  
NRC Document Control Desk