

CONTROL BLOCK:

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01	A	L	B	R	F	2	2	0	0	-	0	0	0	0	0	-	0	0	3	4	1	1	1	1	4			5	
7	8	9				14		15										25	26							30		37	CAT 38
LICENCE CODE						LICENCE NUMBER														LICENCE TYPE									

CON'T

01 REPORT SOURCE L 050000260712148380411849  
7 8 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 During normal operation, while performing SI 4.1.A-8 (RPS - High Water Level in  
0 3 Scram Discharge Tank), LT-85-45G was found to operate in 76 seconds in net  
0 4 response to simulated step change in level. Design criteria specifies a maximum  
0 5 response time of 71 seconds. Technical Specification 3.1.A requires two  
0 6 operable instrument channels per trip system. There was no danger to the health  
0 7 or safety of the public. Redundant level switch LS-85-45E was operable.

0	8	9																					80										
SYSTEM CODE			CAUSE CODE		CAUSE SUBCODE		COMPONENT CODE										COMP. SUBCODE		VALVE SUBCODE														
0	9		I	A	E	X	I	N	S	T	R	U	T	Z																			
7	8		9	10	11	12	13	14	15	16	17	18	19	20																			
(17) LER/RO REPORT NUMBER		EVENT YEAR				SEQUENTIAL REPORT NO.		OCCURRENCE CODE		REPORT TYPE		REVISION NO.																					
8		3		—		0		8		0		/		0		3		X		—		1											
21		22		23		24		25		26		27		28		29		30		31		32											
ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS		ATTACHMENT SUBMITTED		NPRD-4 FORM SUB		PRIME COMP. SUPPLIER		COMPONENT MANUFACTURER																	
C	X	Z	Z	0	0	0	0	0	Y	N	N	R	3	6	9																		
33	34	35	36	37	38	39	40	41	42	43	44	45	46	47	48																		

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 Cause of transmitter response is not known. The Rosemount 1153 transmitter (LT-  
1 1 85-45G) was replaced. Calibration and functional test was performed and the  
1 2 system returned to normal. Rosemount evaluated the transmitter and found no  
1 3 problem with the cell isolators. The transmitters on the SDIV will be replaced  
1 4 with Fluid Components Incorporated level switches.

FACILITY STATUS (1) 5 (2) E (3) 0 (4) 9 (5) 3 (6) 29 (7) NA (8) 40  
 % POWER (9) 0 (10) 9 (11) 3 (12) 29 (13) NA (14) 40  
 OTHER STATUS (15) 40  
 METHOD OF DISCOVERY (16) B (17) 31 (18) Surveillance Testing (19) 32  
 DISCOVERY DESCRIPTION (20) 32  
 ACTIVITY CONTENT (21) 1 (22) 6 (23) Z (24) 33 (25) Z (26) 34 (27) NA (28) 44  
 RELEASED OR RELEASE (29) 44  
 AMOUNT OF ACTIVITY (30) 44  
 LOCATION OF RELEASE (31) 45  
 PERSONNEL EXPOSURES (32) 45  
 NUMBER (33) 1 (34) 7 (35) 0 (36) 0 (37) 0 (38) Z (39) 38 (40) NA (41) 46  
 TYPE (42) 46  
 DESCRIPTION (43) 46  
 PERSONNEL INJURIES (44) 46  
 NUMBER (45) 1 (46) 8 (47) 0 (48) 0 (49) 0 (50) 40 (51) NA (52) 47  
 DESCRIPTION (53) 47  
 LOSS OF OR DAMAGE TO FACILITY (54) 47  
 TYPE (55) 1 (56) 9 (57) Z (58) 42 (59) NA (60) 48  
 DESCRIPTION (61) 48  
 PUBLICITY (62) 48  
 ISSUED (63) 2 (64) 0 (65) N (66) 44 (67) NA (68) 49  
 DESCRIPTION (69) 49  
 NRC USE ONLY (70) 49

NAME OF PREPARER G. T. Chambers

PHONE: (205) 729-0626

LER SUPPLEMENTAL INFORMATION

BFRO-50- 260 / 83080 R1 Technical Specification Involved 3.1.A

Reported Under Technical Specification 6.7.2.b.(2) \* Date Due NRC 05/01/84

Event Narrative:

Units 1 and 3 were in refueling outages. Unit 2 was at 93 percent power. Only unit 2 was affected by this event. During performance of SI 4.1.A-8 (Reactor Protection System - High Water Level in Scram Discharge Tank) LT-85-45G was found to operate in 76 seconds in response to a simulated step change in level. Design criteria calls for a step change response in a maximum time of 71 seconds. Technical Specification 3.1.A requires two operable instrument channels per trip system and this event reduced that number to one in "A" trip system. "B" trip system was also available and operable.

Cause of slow transmitter response is not known. The Rosemount 1153 transmitter LT-85-45G was replaced, the new transmitter was calibrated, functionally tested and returned to service. There was no danger to the health or safety of the public. Redundant level switch LS-85-45E was operable.

The transmitter and remote seal unit was taken apart upon Rosemount's request and the transmitter was shipped to the Rosemount factory. Transmitter LT-85-45G (SN 404251) was evaluated and no problems were found with the cell isolators. (Letter from Rosemount to TVA attached.) Since the unit was disassembled for shipping, no further tests can be performed on the transmitter and remote seal as a unit. The Rosemount transmitters on unit 2 SDIV will be replaced with Fluid Components Incorporated level switches during refueling outage cycle 5 (ECN P0392).

Previous Similar Events

BFRO-50-260/83063

Retention: Period - Lifetime; Responsibility - Document Control Supervisor

\*Revision: JRP

Stamp Recd (4/2/84)  
and send to Jete Room

PROJECTS DEPT.  
RECEIVED

MAR 12 1984



ROSEMOUNT INC., 12001 WEST 78th STREET EDEN PRAIRIE, MINNESOTA 55344  
Mailing Address: P.O. BOX 35129 MINNEAPOLIS, MINNESOTA 55435  
TEL (612) 941-5560 TWX 910-576-3103 TELEX 29-0183

March 8, 1984

APR 02 1984

Browns Ferry Nuclear Station  
Instrumentation Department  
P.O. Box 2000  
Decatur, AL 35602

Attention: Mr. Ron Burns  
Maintenance Superintendant  
Subject: P.O. # 82PJ1-829142

Gentlemen:

Reference my letters to you dated December 14, 1983 and December 1983. Rosemount has completed the evaluation on the three transmitters (s/n's 404251, 404252) returned to us and the following summarizes our findings. Serial Number 404253 was not returned to Rosemount and conversations with TVA indicates this unit was misplaced at TVA. Therefore, Rosemount has not taken any action relative to this transmitter.

Rosemount received the three subject 1153DB4N0005 Pressure Transmitters (without flanges and remote seals) and performed an evaluation on the isolator diaphragm area to verify their condition. The cell isolators were first cleaned with Methanol and were then examined under 6X, 25X, and 50X magnification. Certain minute dents and scratches were observed (as we would expect) but none of the isolators were punctured. We would expect to observe some minute scratches and dents around the cell isolator under magnification, this is due to inserting the process o-ring and handling. Based on our findings, we have concluded that the three transmitters in question are no longer suspect to isolator punctures and are perfectly acceptable for usage.

Because of the inconvenience this situation may have created, Rosemount will be replacing the four transmitters (s/n's 404249, 404251, 404252, 404250) with new units at no-charge to TVA. Based on conversations with Mr. Mike Duncan, we will be supplying 4-1153DB4PG transmitters with 8-1159C25B remote seals to be delivered within 16-20 weeks. These units are essentially identical to those originally supplied.

Subj	
Act. Per. Subj. (S/N)	
Inspection	
Results	
Security	
Training	
Post. Mod. Test	
Medical Service	
Act. Per. Subj. (S/N)	
Instrumentation	
Measurement	
Control	
Health Physics	
Power Station	
Isolator	
PSO	
Quality Assurance	
Compliance	
Minor Services	
Quality Assurance	
Compliance	
Minor Services	

Browns Ferry Nuclear Station

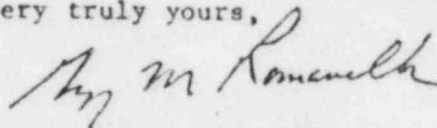
March 8, 1984

Page 2

To reiterate what was said in my letter of 12/14/83. To prevent this situation from recurring, all nuclear remote seals are now assembled in the nuclear manufacturing area. New procedures have been developed and implemented for our nuclear transmitters with remote seals.

We apologize for any inconvenience this matter may have created. We at Rosemount certainly value TVA as a most favored customer. If you have any questions, please call me at (612) 828-3286.

Very truly yours,



Gregory M. Romanchuk  
Applications Engineer

/jka

cc: Jim Braden #41

~~\_\_\_\_\_~~  
Mike Duncan (TVA)

Bob Dhyse #3

Bob Irby (TVA)

TENNESSEE VALLEY AUTHORITY

Browns Ferry Nuclear Plant  
P. O. Box 2000  
Decatur, Alabama 35602

April 11, 1984

Mr. James P. O'Reilly, Regional Administrator  
U. S. Nuclear Regulatory Commission  
Region II  
101 Marietta Street, Suite 3100  
Atlanta, Georgia 30303

Dear Mr. O'Reilly:

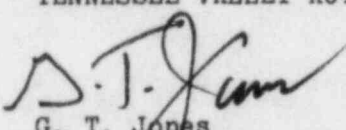
TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 2 - DOCKET  
NO. 50-260 - FACILITY OPERATING LICENSE DPR-52 - REPORTABLE OCCURRENCE  
REPORT BFRO-50-260/83080 R1

Reference: H. L. Abercrombie's memorandum to you dated January 12, 1984

The enclosed report provides followup information concerning LT-85-45G  
found to operate in 76 seconds in net response to simulated step change in  
level when design criteria specifies a maximum response time of 71 seconds.  
This report is submitted in accordance with Browns Ferry Unit 2 Technical  
Specification 6.7.2.b.(2).

Very truly yours,

TENNESSEE VALLEY AUTHORITY

  
G. T. Jones  
Power Plant Superintendent  
Browns Ferry Nuclear Plant

Enclosure

cc (Enclosure):  
U. S. Nuclear Regulatory Commission  
Document Control Desk  
Washington D.C. 20555

NRC Inspector, Browns Ferry Nuclear Plant