



**Wisconsin Electric** POWER COMPANY  
231 W. MICHIGAN, P.O. BOX 2046, MILWAUKEE, WI 53201

April 23, 1984

Mr. J. G. Keppler, Regional Administrator  
Office of Inspection and Enforcement,  
Region III  
U. S. NUCLEAR REGULATORY COMMISSION  
799 Roosevelt Road  
Glen Ellyn, Illinois 60137

Dear Mr. Keppler:

DOCKET NOS. 50-266 AND 50-301  
RESPONSE TO NRC INSPECTION REPORT NOS.  
50-266/84-02 AND 50-301/84-01  
POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2

In response to the subject inspection reports dated March 26, 1984, the following is provided pursuant to 10 CFR 2.201:

1. Containment High-Range Radiation Monitors

Our procedure HPCAL 3.25 has been revised to require the recording on a specific form of the results of the high-range source calibration performed on a refueling basis. Similarly, procedure HPIP 7.51A has been revised to require the recording on a specific form of the results of the internal check source calibration performed on a monthly basis. These corrective actions have been completed. Calibration checks made subsequent to the inspection have verified the correct operability of the monitors during the period in question.

2. Portable Shielding

We have reviewed the portable shielding in the vicinity of the control room and the C-59 area and have the following observations:

- a. Based on subsequent work on the temporary shielding, we believe a small crew (two or three men) could have mounted the curtains for the control room in about two to four hours, not the 18-25 hours originally estimated.

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- b. The major theoretical contribution to accident dose is from a semiinfinite gas cloud formed by containment leakage of 0.4% per day. However, design calculations conservatively ignore the time required for the buildup of such a cloud. Hence, in reality, the dose received in the enclosed turbine hall around the control room in the first hour or two following an accident is likely to be small. Accordingly, in the period from January 1 to January 6, there was little risk, if any, to plant staff due to the short interval needed to mount the curtains provided.
- c. Our inventory of the curtains indicated that only one half of the design thickness was in place at the control room due to a misinterpretation of the original design drawings. This was rectified on April 5. An analysis of the decrease in shielding indicated a theoretical accident dose of 5.7 Rem. However, a minor modification of a conservative occupancy factor related to time spent at certain distances from the window resulted in a calculated dose within the General Design Criteria (GDC) of 5 Rem to the whole body. The revised assumption is more realistic, yet still conservative.
- d. The racks have now been stenciled to assure their correct orientation if needed. All curtains are hung and the curtains used at the control room have been stenciled to indicate their equivalent densities.
- e. As your inspectors are aware, certain of the racks and curtains in the C-59 area have been shortened to provide overhead clearance and, thereby, facilitate their positioning in the event of an accident. Thirty-six curtains are out for shortening at the present time and will be returned on or before May 12. The number of these curtains is for our convenience in use and habitability criteria are met even during periods when batches are returned to the manufacturer for modification. This discussion is provided for your information.

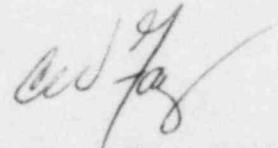
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- f. In summary, while our implementation of this portable shielding was, for brief periods, less conservative than we originally intended, we believe no significant compromise of GDC or other habitability criteria occurred. At the present time, the shielding is in its intended configuration, except for the convenience shortening of curtains which will be completed on or before May 12.

Very truly yours,

A handwritten signature in dark ink, appearing to read 'C. W. Fay', with a stylized flourish at the end.

Vice President-Nuclear Power

C. W. Fay

Copy to NRC Resident Inspector

