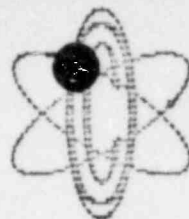


OYSTER CREEK

PHONE 602 • 601-5551



JERSEY CENTRAL POWER & LIGHT COMPANY

NUCLEAR GENERATING STATION

P.O. BOX 388 • FORKED RIVER • NEW JERSEY • 08731

October 11, 1973

Mr. James P. O'Reilly  
Directorate of Regulatory Operations  
Region 1  
U. S. Atomic Energy Commission  
631 Park Avenue  
King of Prussia, Pennsylvania 19406

Dear Mr. O'Reilly:

Subject: Oyster Creek Nuclear Generating Station  
Docket No. 50-219  
Preliminary Abnormal Occurrence Report No. 73-26

Per a telephone conversation between J. L. Sullivan, Jr., and  
D. Caplan on October 10, 1973, we are reporting the attached event as  
an abnormal occurrence, although it is not clear that it is reportable.

Technical Specification 4.1, Table 4.1.1, Note 2, states, "At  
least daily during reactor power operation, the reactor neutron flux  
peaking factor shall be estimated and the flow-referenced AFRM scram  
and rod block settings shall be adjusted, if necessary, as specified in  
Section 2.3, Specifications (1) (a) and (2) (a)." This estimate was,  
in fact, performed as specified and corrections were made as required.

Very truly yours,

*Preliminary*

*J. L. Sullivan, Jr.*  
FOR  
J. L. Sullivan, Jr.

Station Superintendent

JTC/pd

cc: A. Giambusso

*Rec'd  
50219*

8304200612 731011  
PDR ADOCK 05000219  
S PDR

Date: 10/6/73

Time: 2:00 p.m.

Abnormal Occurrence

Report No. 73-26

Failure to set the Average Power Range Monitor scram and red  
black set points to the conservative values specified in Tech-  
nical Specifications 2.3(1)(a) and 2.3(2)(a).

This event is considered to be an abnormal occurrence as defined  
in the Technical Specifications, paragraph 1.15A. Notification  
of this event, as required by the Technical Specifications, para-  
graph 4.6.2(a), was made to AEC Regional Director of Regulatory  
Operations by telephone on October 10, 1973, at 4:30 p.m., and  
by telecopier on October 11, 1973, at 1:15 p.m.

SITUATION: On October 6, 1973, at 2:00 p.m., the reactor startup to full  
power had been halted due to a lack of in-service condensate de-  
mineralizers. The core thermal output at this time was approxi-  
mately 567 MWt and the recirculation flow rate was  $30 \times 10^6$  lbm/hr.  
At this time the Maximum Total Peaking Factor (PF) was estimated  
to be 4.54 and the Average Power Range Monitors (APRM's) were  
set conservatively such that 100% on the APRM's corresponded to  
1200 MWt. This is equivalent to reducing the neutron flux scram  
by the amount  $3.01/\text{PF}$  as specified in Technical Specification  
2.3.1.a, with some added margin. The 100%/1200 MWt setting  
allows for a neutron flux peaking up to a value of 4.84.

October 6, 1973

SITUATION - Continued

At 5:30 p.m., after a heat balance calculation, the setting of the APRM's was inadvertently set such that 100% of the APRM's corresponded to 1400 MWt which accounts for peaking factors of only 4.15. Thus, the limiting safety system setting for the APRM Neutron Flux Scram and rod block were set less conservatively than specified in the Technical Specification 2.3.1.a and 2.3.2.a.

CAUSE:

An investigation is yet to be conducted to determine the exact cause of this occurrence. However, at this time, it is believed that it was caused by a communication problem.

REMEDIAL ACTION:

At 10:00 a.m. on October 7, 1973, the reactor neutron flux peaking factor was estimated as required in Technical Specification 4.1, Table 4.1.1, Note 2, and found to be 4.71. The APRM's were then correctly adjusted to the conservative 100%/1200 MWt setting.

SAFETY SIGNIFICANCE:

Based on the Neutron Flux Peaking Factor of 4.71, as estimated at the time of the correction, the safety limit can be shown to be at 1228 MWt for the recirculation flow rate of  $30 \times 10^6$  lbm/hr. Using the 100%/1400 MWt setting of the APRM's, the reactor at this condition would have scrammed at 1200 MWt, if required.

Thus, the safety limit would not have been exceeded.

Reviewed by: *H. C. Heffernan*

Date: *10/11/73*

## MEMO ROUTE SLIP

Form AEC-93 (Rev. May 14, 1947) AECM 0240

☐ See me about this.☐ For concurrence.☐ For action.☐ Note and return.☐ For signature.☐ For information.

TO (Name and unit)	INITIALS	REMARKS
H. D. Thornburg, Chief, FS&EB	DATE	Licensee: Jersey Central Power & Light Co.
		Docket No.: 50-219
		Abnormal Occurrence: AO 73-26
TO (Name and unit)	INITIALS	REMARKS
RO:HQ (5) DR Central Files (1) Regulatory Standards (3) Dir. of Licensing (13)	DATE	The attached report from the subject licensee is
		forwarded in accordance with RO Manual Chapter 1000.
TO (Name and unit)	INITIALS	REMARKS
RO Files	DATE	The action taken by the licensee is considered
		appropriate. Followup will be performed during
		the next inspection as appropriate. Copies of
FROM (Name and unit)	REMARKS	
<i>D. L. Caphton</i> D. L. Caphton, Senior Reactor Inspector, BWR Section		the report have been forwarded to the PDR, Local
		PDR, NSIC, DTIE and State representatives. The
		licensee will submit a 10 day written report to
PHONE NO.	DATE	Licensing.
	10/11/73	

USE OTHER SIDE FOR ADDITIONAL REMARKS

GPO : 1971 O - 445-469