JERSEY CENTRAL POWER & LIGHT COMPANY

OYSTER CREEK NUCLEAR GENERATING STATION

PHONE 609 • 693-6651

P.O. BOX 388 • FORKED RIVER • NEW JERSEY • 08731

October 11, 1973

Mr. James P. O'Reilly
Directorate of Regulatory Operations
Region 1
U. S. Atomic Energy Commission
631 Park Avenue
King of Prussia, Pennsylvania 19406

Dear Mr. O'Reilly:

Subject: Oyster Creek Nuclear Generating Station
Docket No. 50-219
Preliminary Abnormal Occurrence Report No. 73-26

Per a telephone conversation between J. L. Sullivan, Jr., and D. Caphton on October 10, 1973, we are reporting the attached event as an abnormal occurrence, although it is not clear that it is reportable.

Technical Specification 4.1, Table 4.1.1, Note 2, states, "At least daily during reactor power operation, the reactor neutron flux peaking factor shall be estimated and the flow-referenced APRM scram and rod block settings shall be adjusted, if necessary, as specified in Section 2.3, Specifications (1) (a) and (2) (a)." This estimate was, in fact, performed as specified and corrections were made as required.

Very truly yours,

preliminary

J. L. Sullivan, Jr.
for
T. Carroll, Jr.
Station Superintendent

JTC/pd

cc: A. Giambusso

Docket 50-219

Date: 10/6/73
Time: 2:00 p.m.

Preliminary
Abnormal Occurrence
Report No. 73-26

ITEM: Failure to set the Average Power Range Monitor scram and rod block set points to the conservative values specified in Technical Specifications 2.3(1)(a) and 2.3(2)(a).

This event is considered to be an abnormal occurrence as defined in the Technical Specifications, paragraph 1.15A. Notification of this event, as required by the Technical Specifications, paragraph 6.6.2.a, was made to the Regional Director of Regulatory Operations by telephone on October 10, 1973, at 4:30 p.m., and by telecopier on October 11, 1973, at 1:15 p.m.

SITUATION: On October 6, 1973, at 2:00 p.m., the reactor startup to full power had been halted due to a lack of in-service condensate demineralizers. The core thermal output at this time was approximately 567 MWt and the recirculation flow rate was $30X10^6$ lbm/hr. At this time the Maximum Total Peaking Factor (PF) was estimated to be 4.54 and the Average Power Range Monitors (APRM's) were set conservatively such that 100% on the APRM's corresponded to 1200 MWt. This is equivalent to reducing the neutron flux scram by the amount 3.01/PF as specified in Technical Specification 2.3.1.a, with some added margin. The 100%/1200 MWt setting allows for a neutron flux peaking up to a value of 4.84.

SITUATION - Continued

At 5:30 p.m., after a heat balance calculation, the setting of the APRM's were inadvertently set such that 100% of the APRM's corresponded to 1400 MWt which accounts for peaking factors of only 4.15. Thus, the limiting safety system setting for the APRM Neutron Flux Scram and rod block were set less conservatively than specified in the Technical Specification 2.3.1.a and 2.3.2.a.

CAUSE: An investigation is yet to be conducted to determine the exact cause of this occurrence. However, at this time, it is believed that it was caused by a communication problem.

REMEDIAL ACTION:

At 10:00 a.m. on October 7, 1973, the reactor neutron flux peaking factor was estimated as required in Technical Specification 4.1, Table 4.1.1, Note 2, and found to be 4.71. The APRM's were then correctly adjusted to the conservative 100%/1200 MWt setting.

SAFETY SIGNIFICANCE:

Based on the Neutron Flux Peaking Factor of 4.71, as estimated at the time of the correction, the safety limit can be shown to be at 1228 MWt for the recirculation flow rate of 30×10^6 lbm/hr. Using the 100%/1400 MWt setting of the APRM's, the reactor at this condition would have scrammed at 1200 MWt, if required. Thus, the safety limit would not have been exceeded.

Prepared by: [illegible] Date: 10/6/73

MEMO ROUTE SLIP
Form AEC-93 (Rev. May 14, 1947) AECM 0240

☐ See me about this. ☐ Note and return. ☑ For concurrence. ☐ For signature. ☐ For action. ☐ For information.

TO (Name and unit)	INITIALS	REMARKS
H. D. Thornburg, Chief, FS&EB	DATE	Licensee: Jersey Central Power & Light Co. Docket No.: 50-219 Abnormal Occurrence: AO 73-26
TO (Name and unit) RO:HQ (5) DR Central Files (1) Regulatory Standards (3) Dir. of Licensing (13)	INITIALS DATE	REMARKS The attached report from the subject licensee is forwarded in accordance with RO Manual Chapter 1000.
TO (Name and unit) RO Files	INITIALS DATE	REMARKS The action taken by the licensee is considered appropriate. Followup will be performed during the next inspection as appropriate. Copies of
FROM (Name and unit) D. L. Caphton, Senior Reactor Inspector, BWR Section	REMARKS	the report have been forwarded to the PDR, Local PDR, NSIC, DTIE and State representatives. The licensee will submit a 10 day written report to Licensing.
PHONE NO.	DATE 10/11/73	

USE OTHER SIDE FOR ADDITIONAL REMARKS

GPO : 1971 O - 445-469